



Nuclear Fusion

Research for the energy of the future



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- J. Riemann

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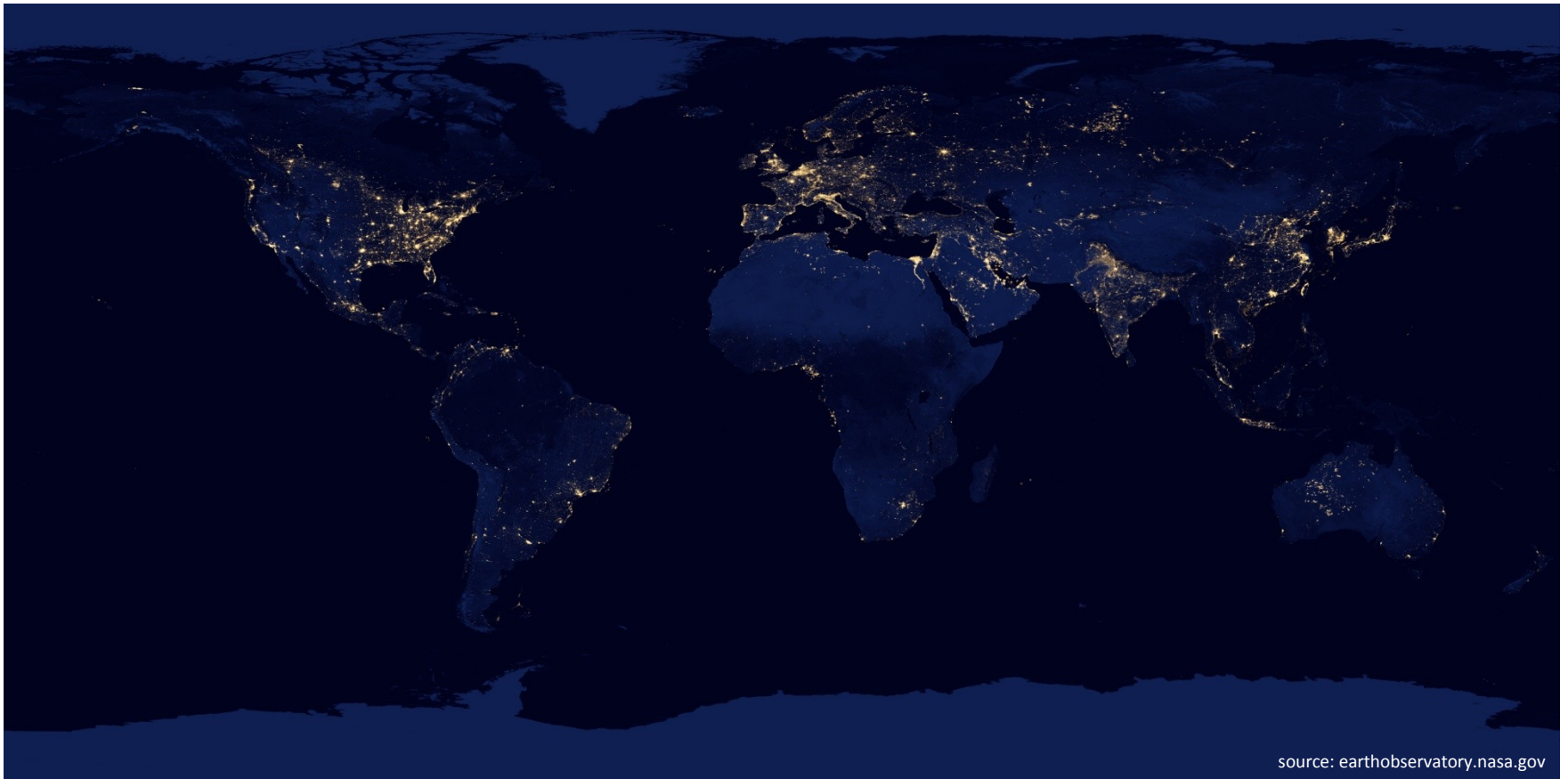
- Max-Planck-Institut für Plasmaphysik (IPP)
- why do we need nuclear fusion?
- physical basics of nuclear fusion
- what is a plasma?
- the path to fusion: Tokamaks and Stellarators
- fusion experiment Wendelstein 7-X
- ITER and the path to a power plant

- institute of the Max-Planck-Gesellschaft (MPG)
- national research laboratory (Helmholtz-Gemeinschaft)
- located in Garching and Greifswald
- 1994: foundation of the Greifswald branch
- 2000: completion of building
- Staff: approximately 450 people
- construction and operation of the fusion experiment W7-X (one of Germany's biggest research projects)
- 10.12.2015: first plasma

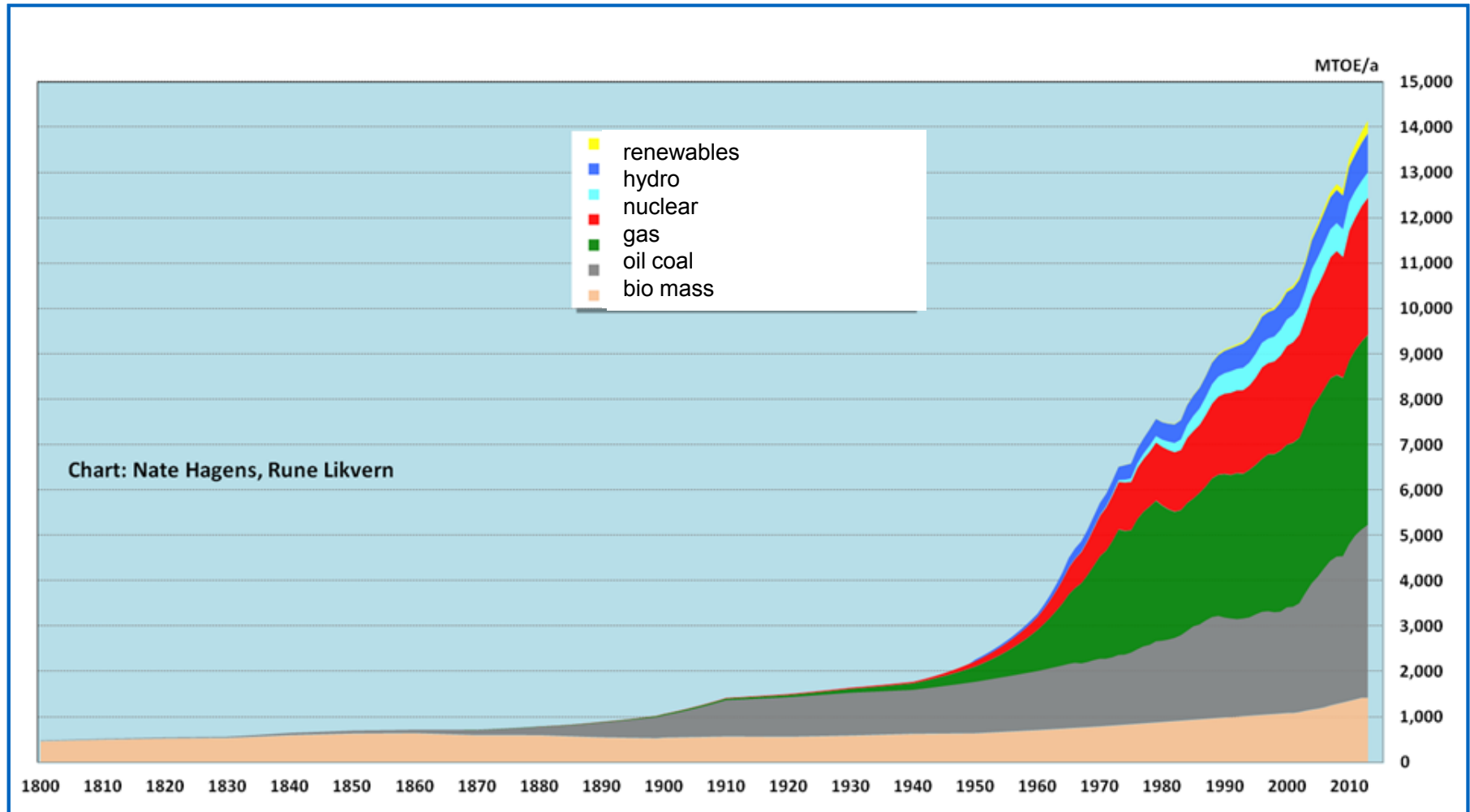
Experimental and theoretical basic research for utilising controlled nuclear fusion based on magnetic confinement for the production of energy

- design, construction and operation of fusion experiments (ASDEX-Upgrade, Wendelstein 7-X)
- development of diagnostic methods
- obtaining data relevant for fusion
- theoretical research in plasma physics
- numerical modelling of fusion plasmas
- research of materials and technology
- contributions to the first fusion reactor (ITER)

WHY DO WE NEED NEW SOURCES OF ENERGY?

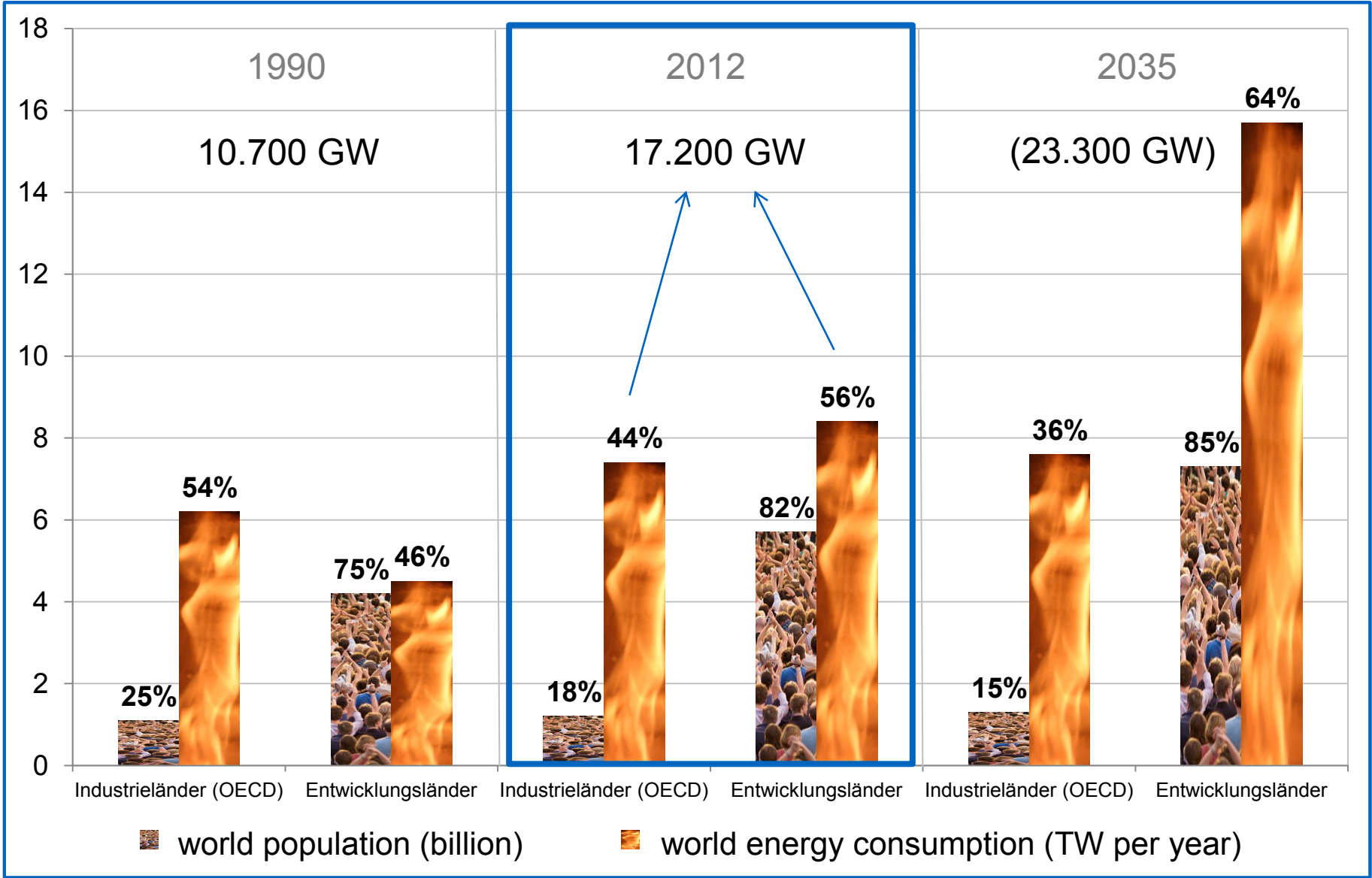


Electricity consumption \approx 1/6 of primary energy consumption

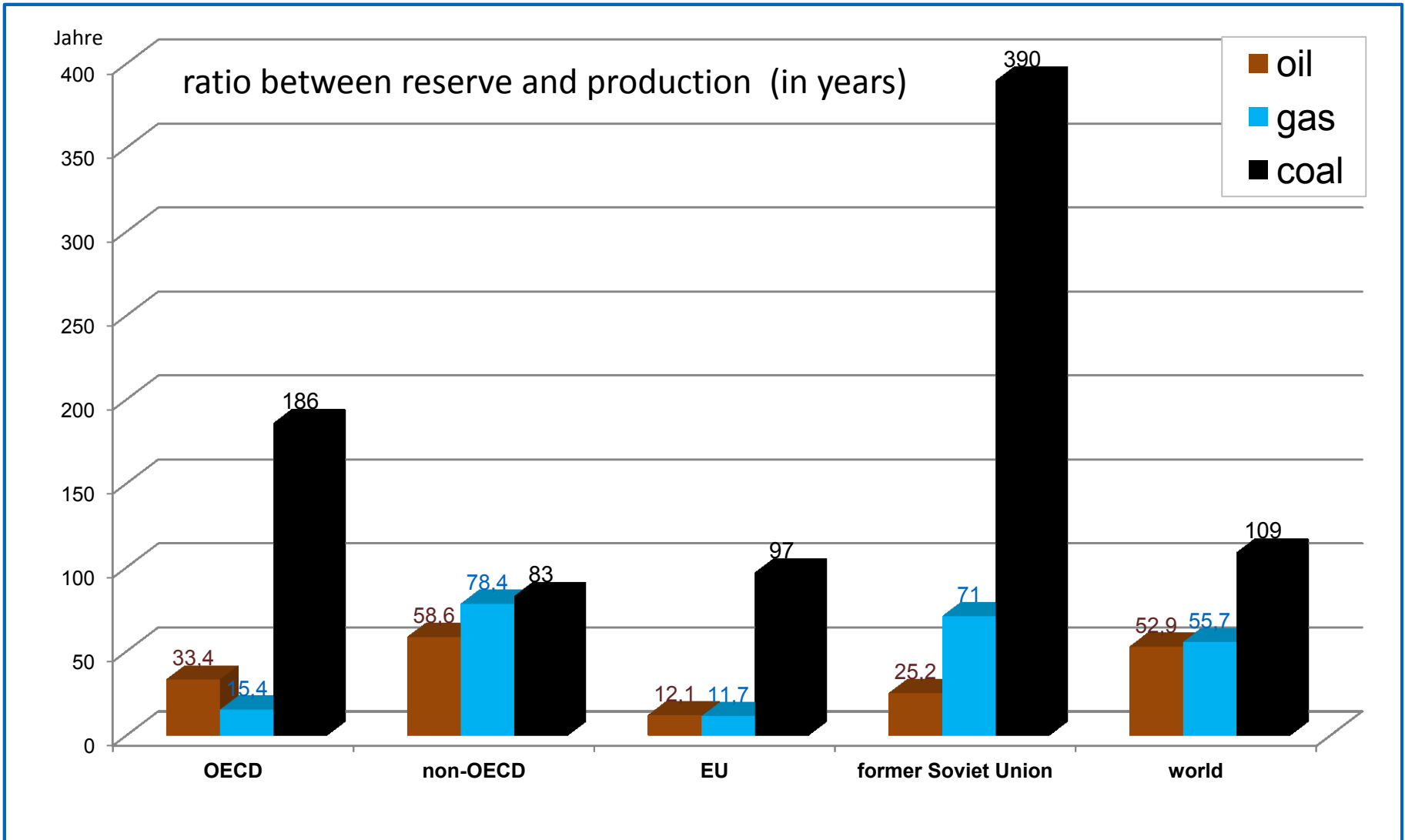


source: Nate Hagens, Rune Likvern

Energy Consumption and World Population



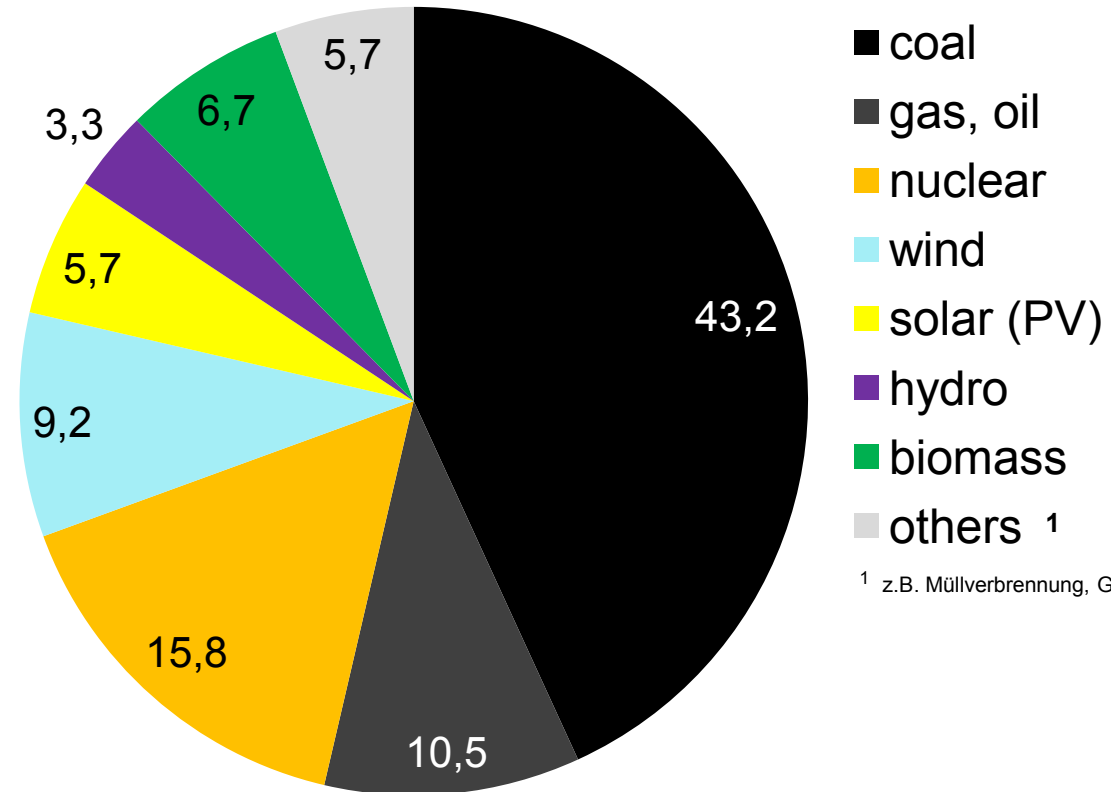
source: BP Energy Outlook 2035, January 2014; UN World Population Prospects, 2010 Revision; A. Kleiber 2014



Reserve-zu-Produktions-Verhältnis (R/P) für die fossilen Energieträger Öl, Gas und Kohle Ende 2012; source: BP Statistical Review of World Energy June 2013, A. Kleiber 2014



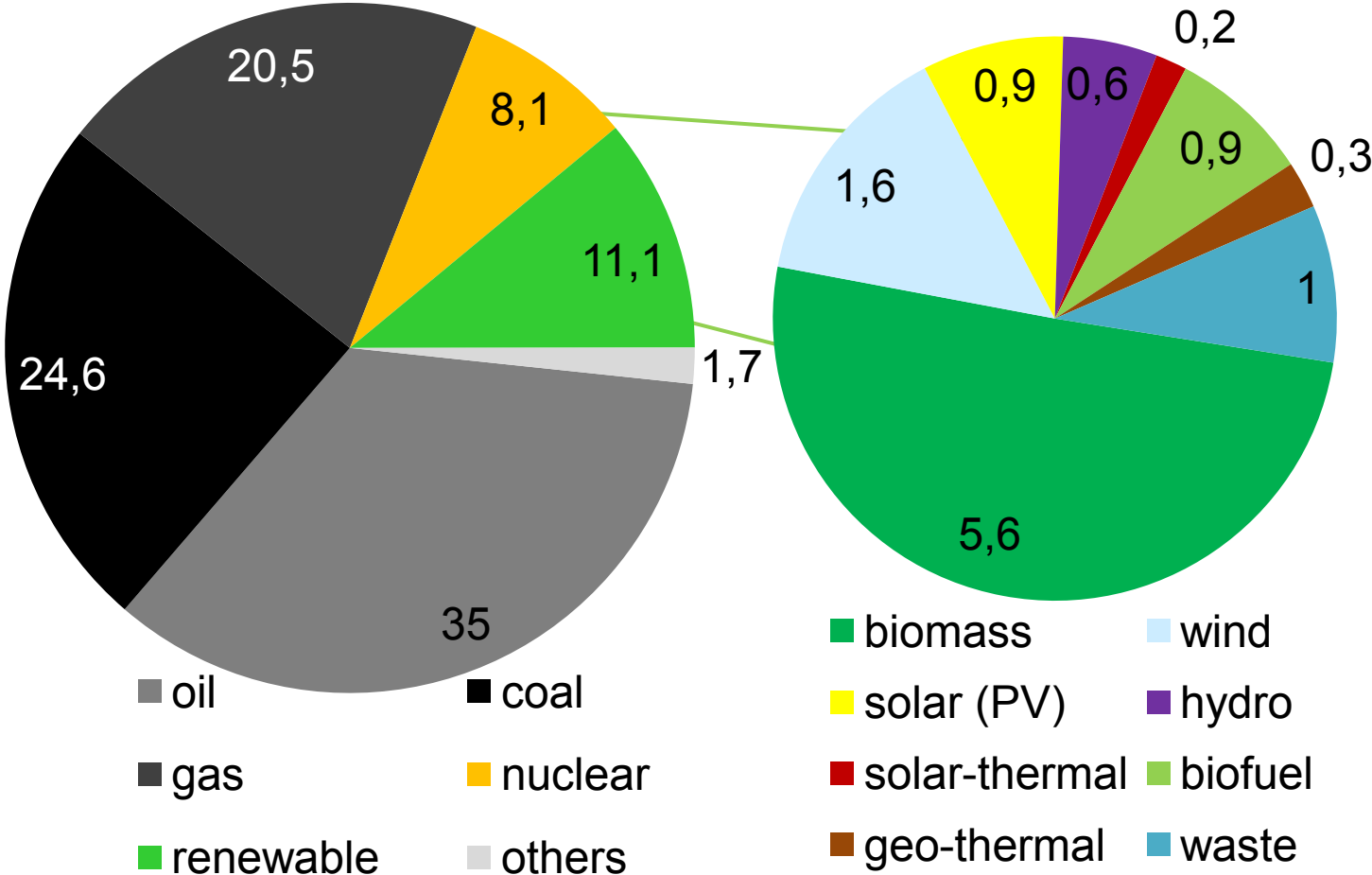
Percentage of 63 GJ



¹ z.B. Müllverbrennung, Geothermie,

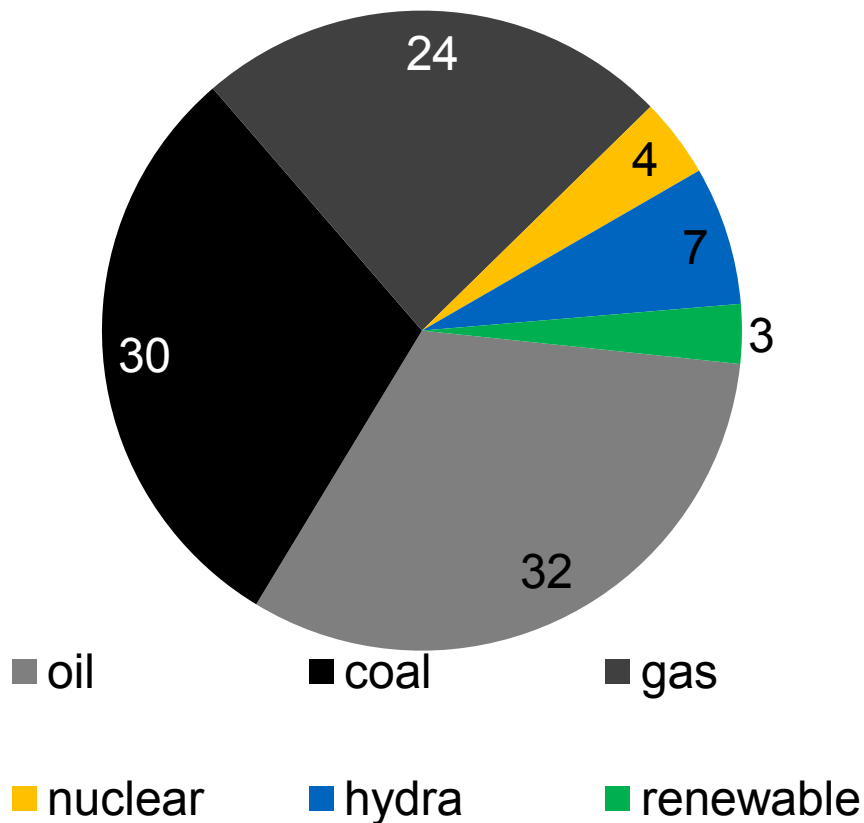
Quelle: AGEb 2015, A. Kleiber 2016

Percentage of 412 GW

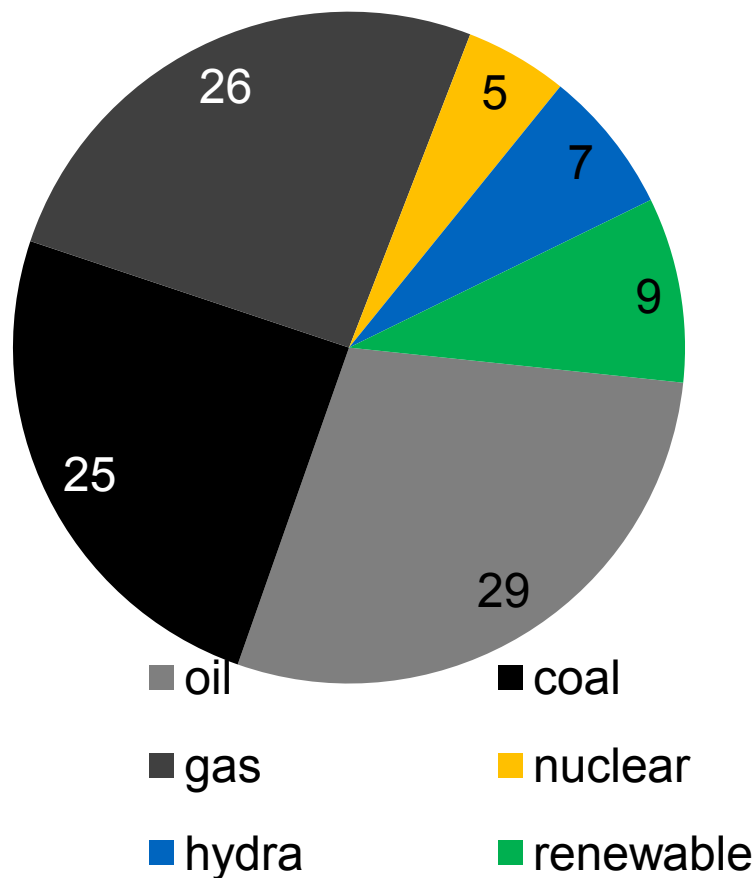


Quelle: AGEB 2015, A. Kleiber 2016

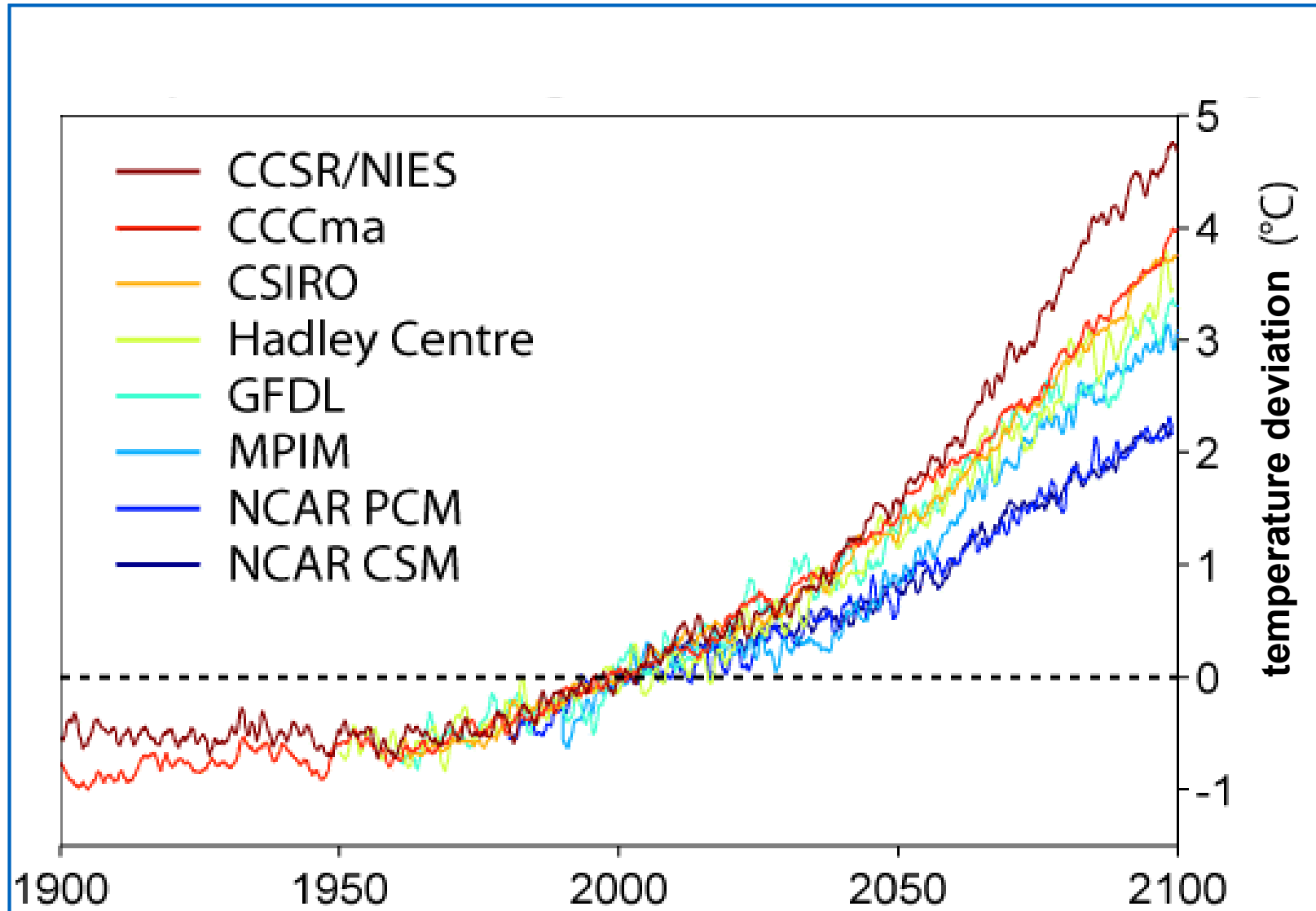
**2014: Percentage of
17 200 GW**



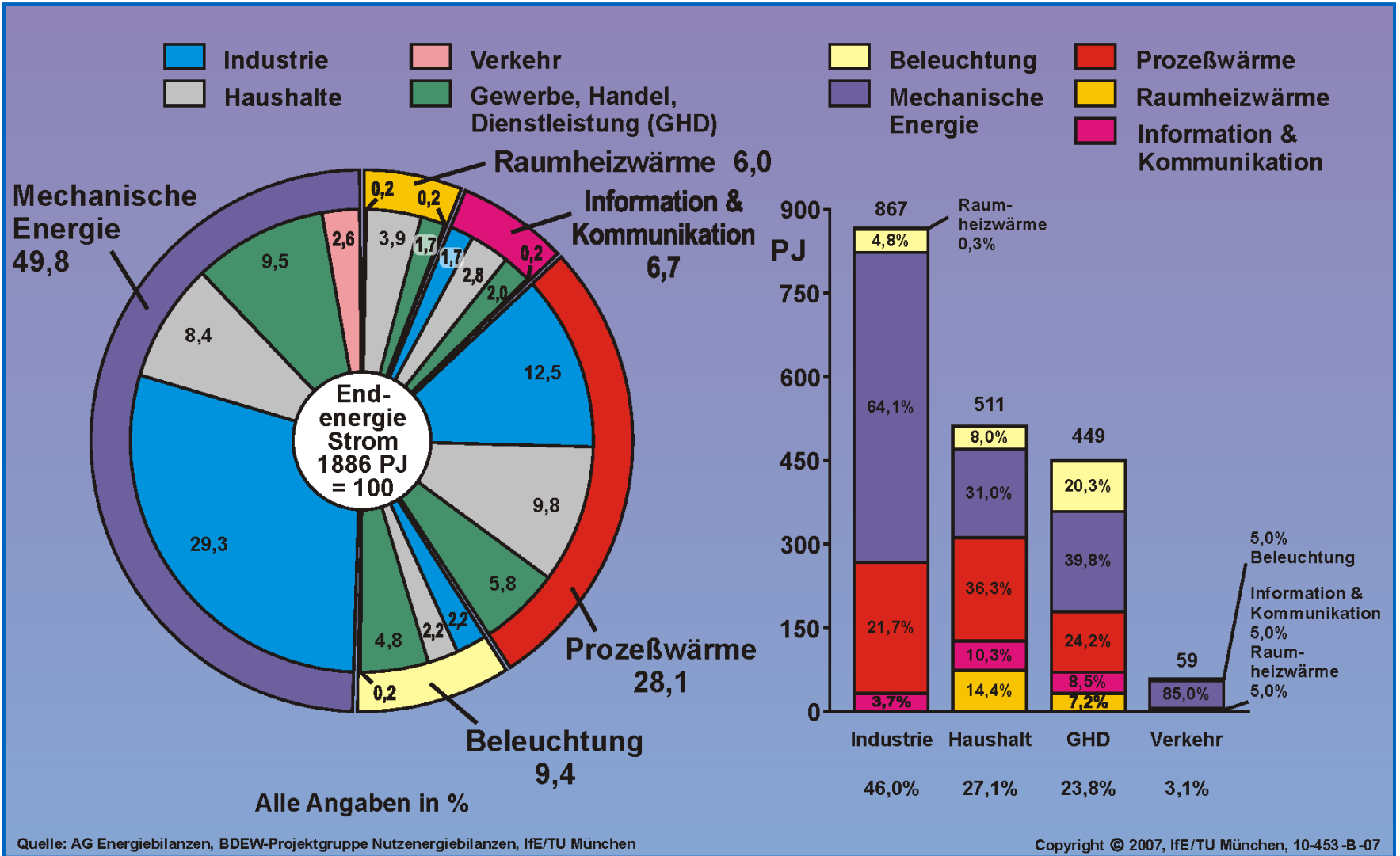
**2035: Percentage of
23 000 GW**



Quelle: BP Energy Outlook 2035, Feb. 2016, A. Kleiber 2016



sources: de.wikipedia „Globale Erwärmung“



Challenges:

- rising energy consumption
- secure energy supply
- limitations of fossile energy (possible substitute?)
- curbing of global warming (CO₂-reduction)

Strategy for minimising risk:

- parallel development of different concepts as part of a future energy supply concept
(solar, wind, storage of energy, nuclear fusion, ...)↑

Example: Berlin

- population: 3,5 million
- area: 900 km²
- electricity demand (2010): 1,5 GW
(10 GW total primary energy)
- wind power (2 W/m²): 80 % city area
- solar power (15 W/m²): 10 % city area
- anthracite: 5 Mt (4 Olympia stadia)
- nuclear fission: 260 t natural Uranium (14 m³)
- nuclear fusion: 0,6 t Deuterium (D) and Tritium (T)

fusion: electricity demand of a family for one year (3000 kWh) covered by 0,08 g D and 0,2 g ⁶Li.



source: M. Borchardt 2014

D in 4,5 l water and Lithium (⁶Li) in an old laptop battery can supply a family with electricity for 3 years.

PHYSICAL BASICS

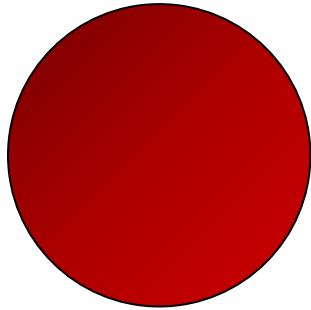
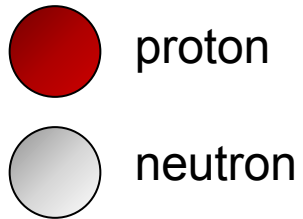
$$\nabla \vec{E} = \frac{1}{\epsilon_0} \rho$$

$$\nabla \times \vec{B} = \mu_0 \vec{j} + \mu_0 \epsilon_0 \frac{\partial \vec{E}}{\partial t}$$

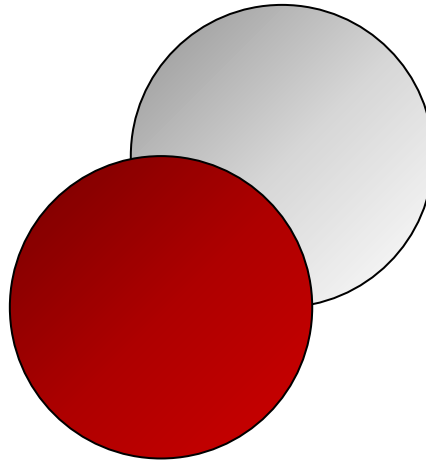
$$\nabla \vec{B} = 0$$

$$\nabla \times \vec{E} = -\frac{\partial \vec{B}}{\partial t}$$

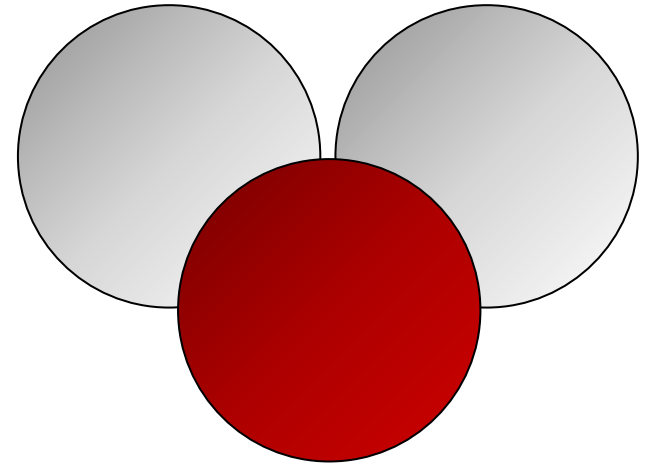
The Three Isotopes of Hydrogen



hydrogen nucleus
H

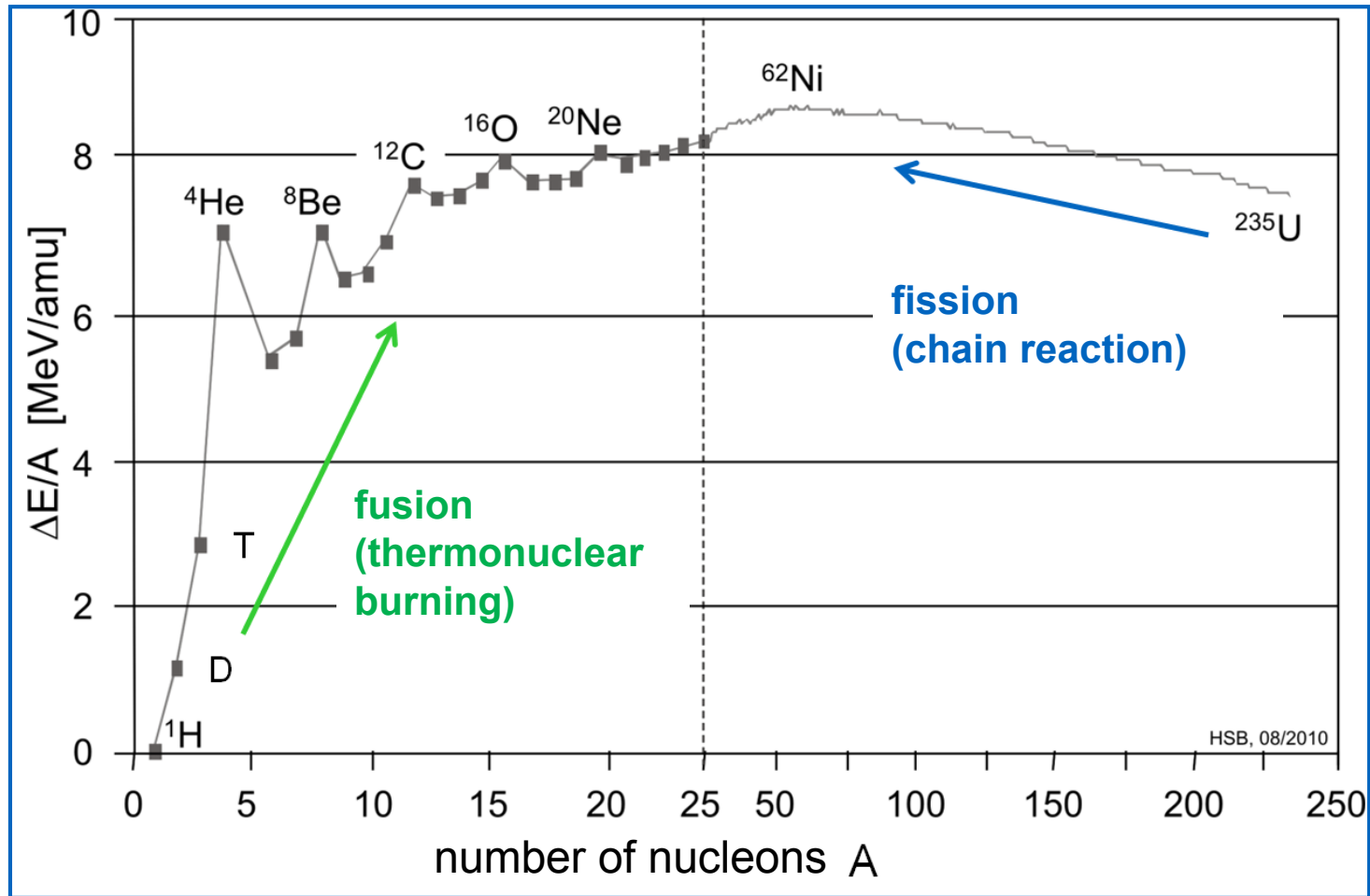


deuterium
D



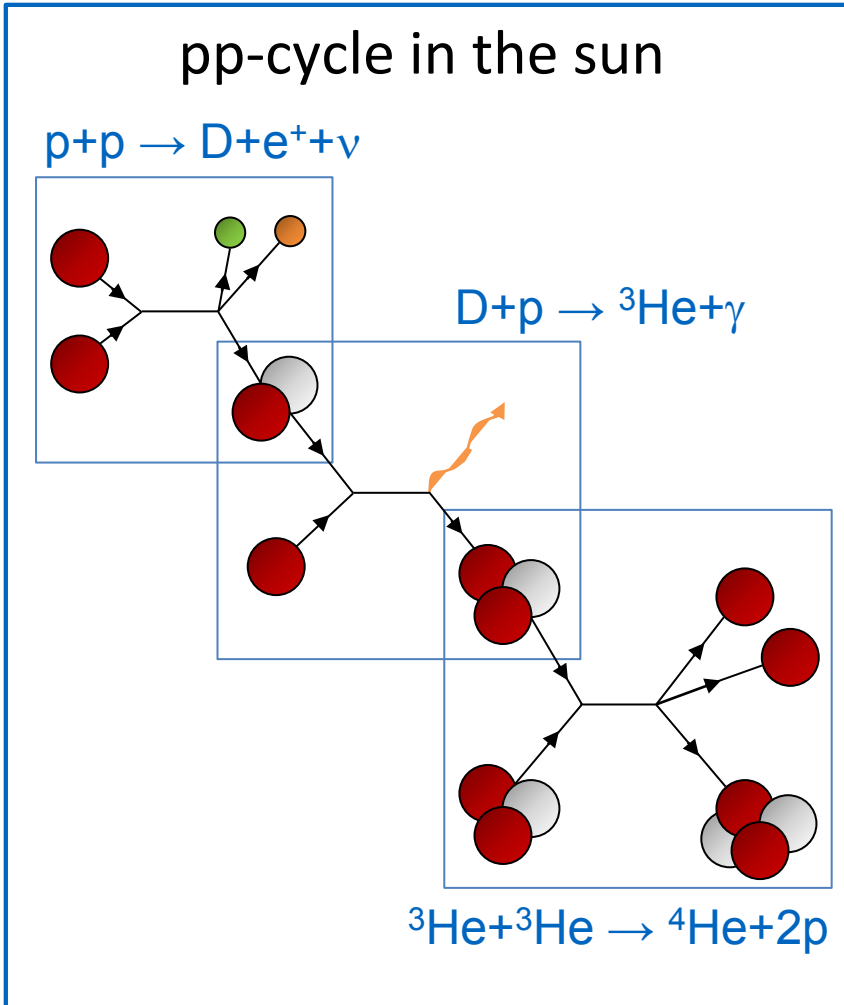
tritium
T

source: J. Riemann 2013

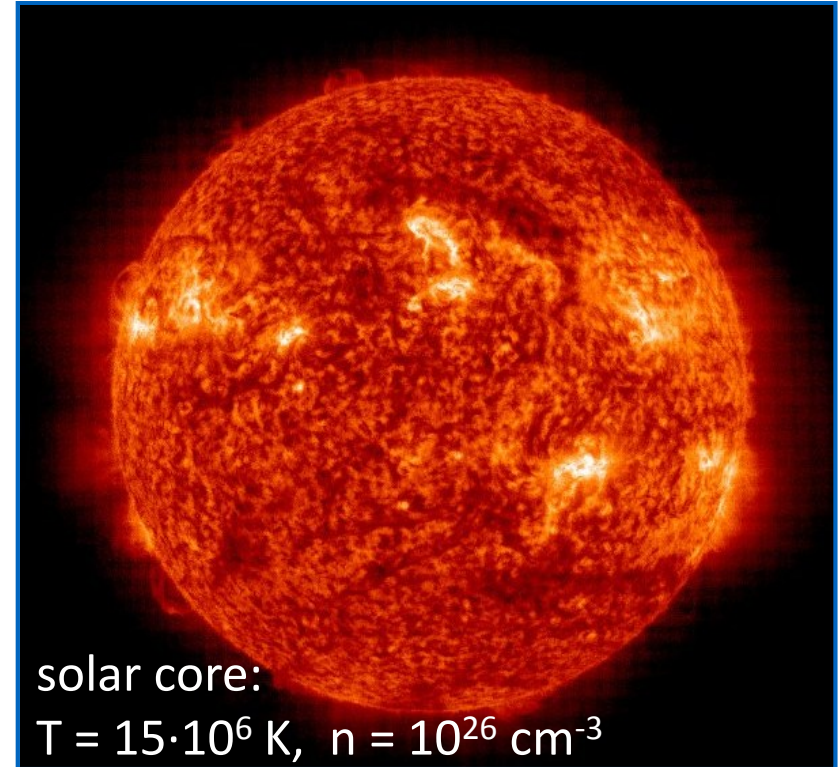


source: H.S. Bosch 2010

Energy can be produced by **fusion of light nuclei** or by **fission of heavy nuclei**



source: A. Kleiber 2014



solar core:

$$T = 15 \cdot 10^6 \text{ K}, \quad n = 10^{26} \text{ cm}^{-3}$$

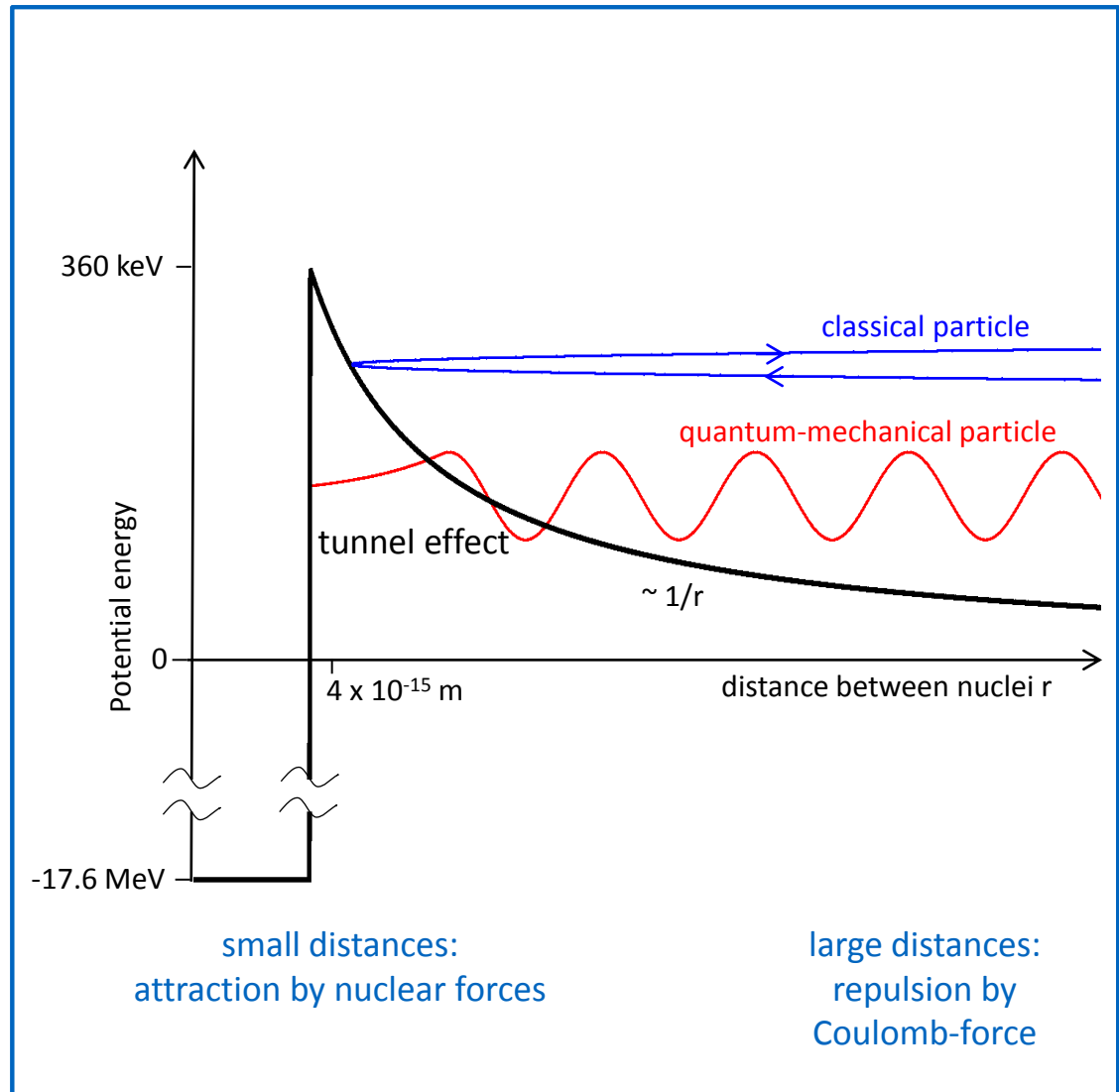
source: SOHO, ESA & NASA (EIT 304) 12. März 2013 um 13.19 UT

Every second 600 Mio. t of hydrogen are transformed into 596 Mio. t of helium according to $E = mc^2$.

This yields a power of $3,6 \cdot 10^{17}$ GW.

Important: Gravitational force overcomes repulsive forces

- potential well: difficult to overcome
- finite probability for tunneling through the barrier
- probability highest for light nuclei with high velocity



source: R. Kleiber 2014

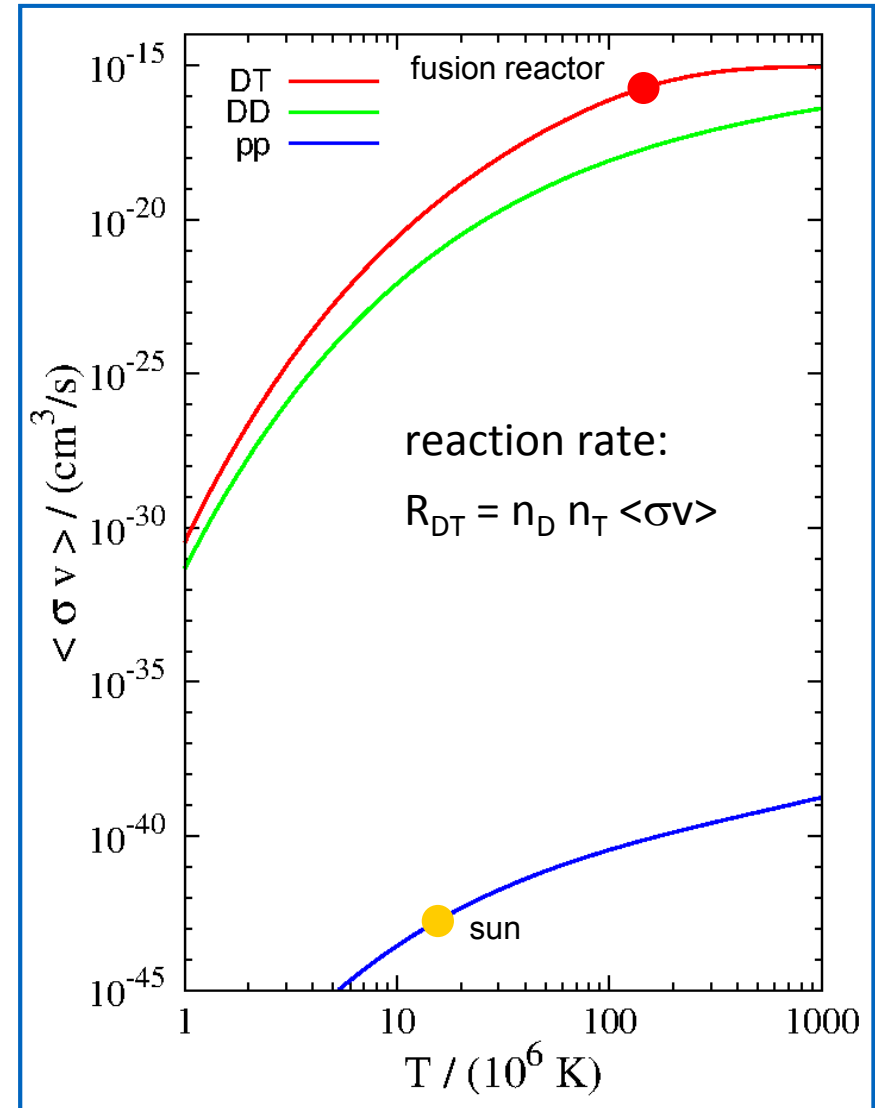
■ low cross-section of pp-cycle

- very slow ($\tau = 3 \cdot 10^9$ a)
- very unlikely
- long life-time of the stars
- not feasible on earth!

■ more likely reaction:

Deuterium-Tritium (DT)

- faster than pp-cycle ($\tau = 100$ s)
- required temperature:
approx. 150 Mio K (15 keV)



source: IPP

Fuels

Deuterium (D)

- part of seawater (0,15 ‰)

Tritium (T)

- can be “bred” from lithium
- radioactive (half-life period 12 a)
- only small amount in reactor due to breeding

Lithium (Li)

- contained in rocks (0,05 ‰)

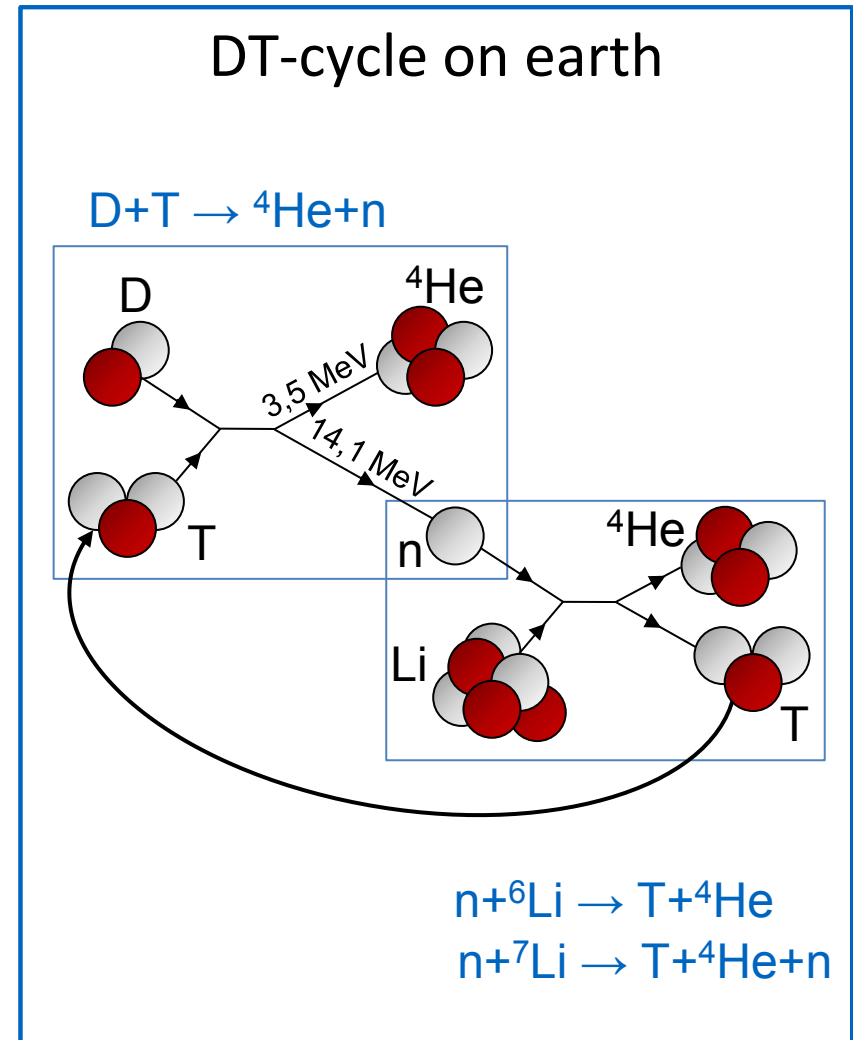
→ resources (D, Li) available for many millenniums

“Ash”

Helium (He)

Energy

kinetic energy of neutrons → heat



DT-fusion reactions, source: A. Kleiber 2014

calory content of a whole
milk chocolate bar (100 g)

⇒ 527 kcal = $1,4 \cdot 10^{19}$ MeV =
1 chocolate unit (CU)

kinetic energy of a car

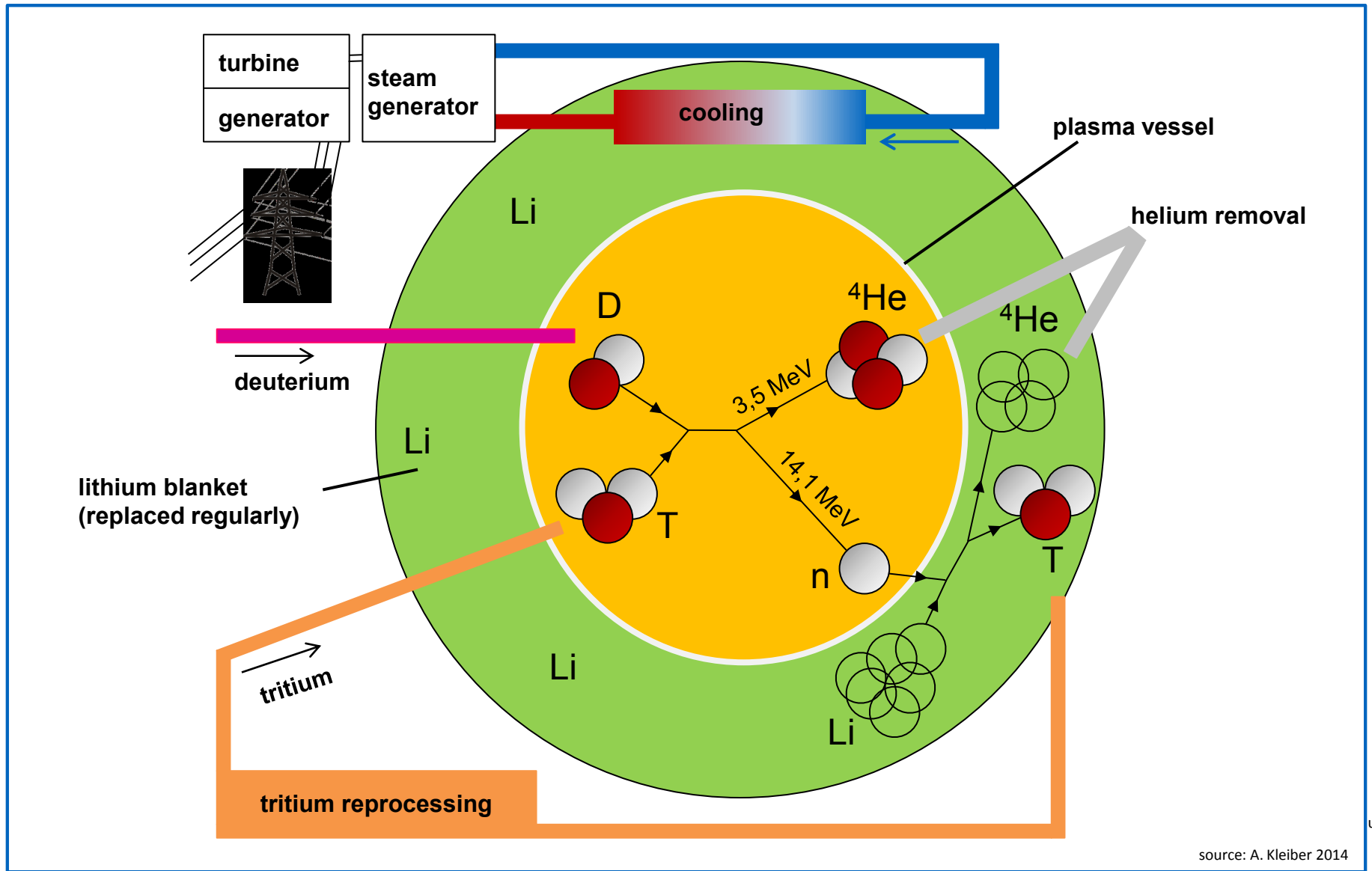
$m = 1$ t, $v = 100$ km/h

⇒ 1/6 CU



source: R. Kleiber 2012

Schematic Diagram of a Fusion Reactor



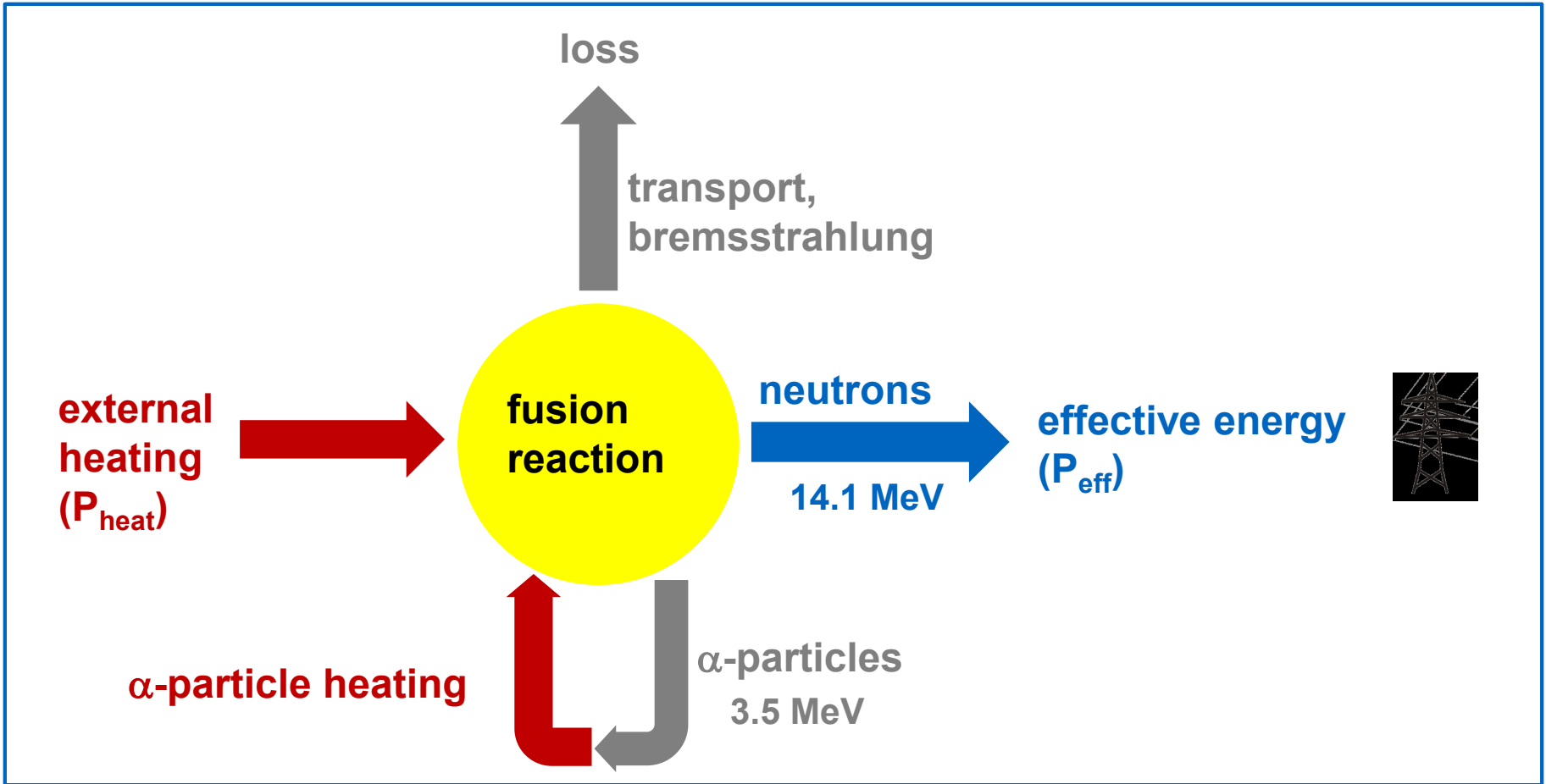
source: A. Kleiber 2014

- high temperature
 - ➔ $T \approx 150 \text{ Mio K}$
- sufficient number of particles per volume (density)
 - ➔ $n \approx 10^{20} \text{ m}^{-3}$
 - (equivalent to 1/500.000 of the atmospheric density)
- good confinement(= heat insulation)
confinement time
(time of cooling down of the plasma after heating)
 - ➔ $\tau_E \approx 5 \text{ s}$



source: M. Borchardt 2014

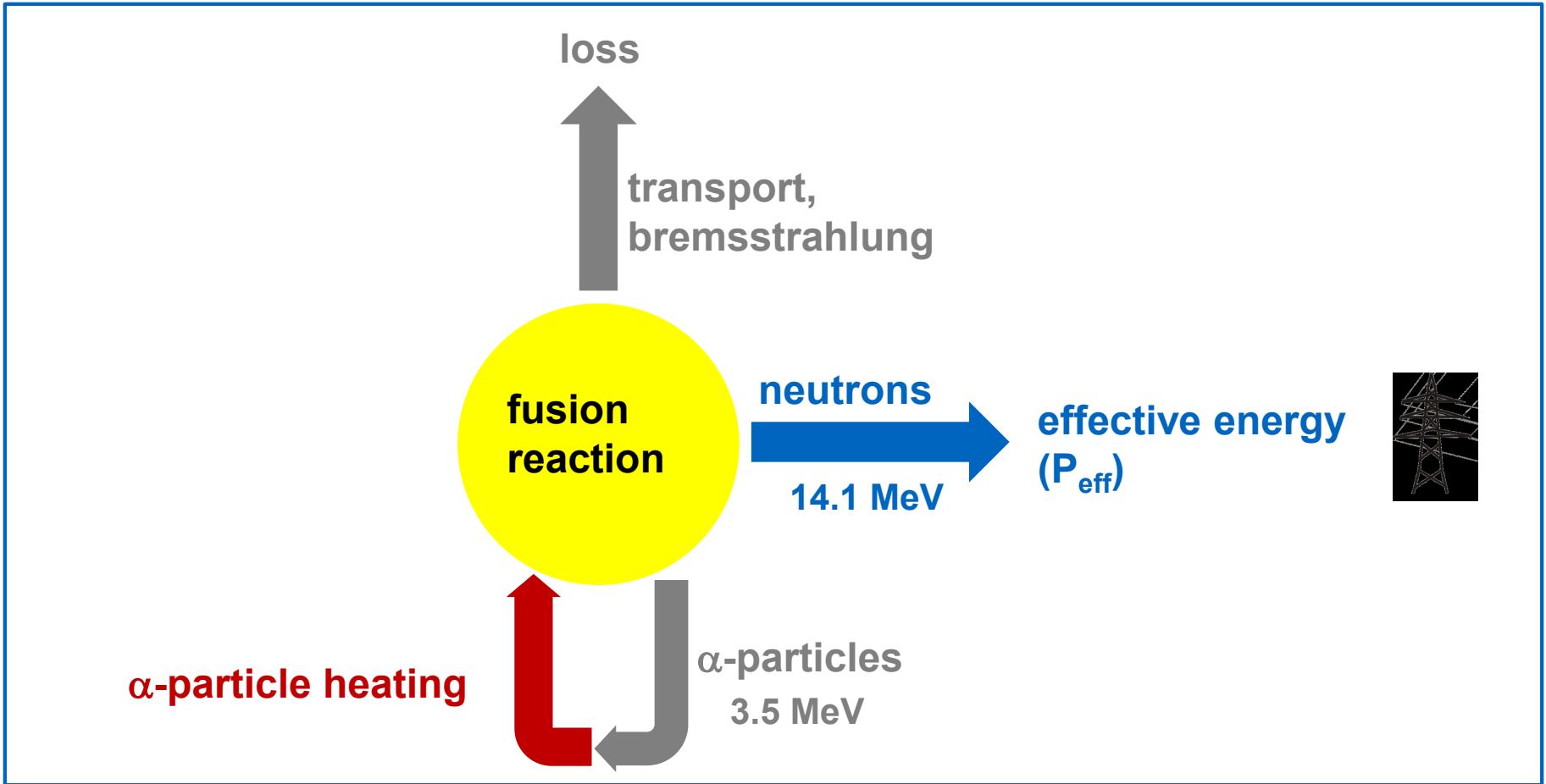
triple product ($n\tau_E T$) has to be sufficiently large to ignite the plasma



source: R. Kleiber 2012

power gain factor: $Q = P_{\text{eff}} / P_{\text{heat}}$

energy gain: $Q > 1$



source: R. Kleiber 2012

$n\tau T$ large enough \rightarrow burning plasma: $Q = \infty$

- neutrons leave the plasma
 → power generation
- α -particles confined
 → heating

- heating:

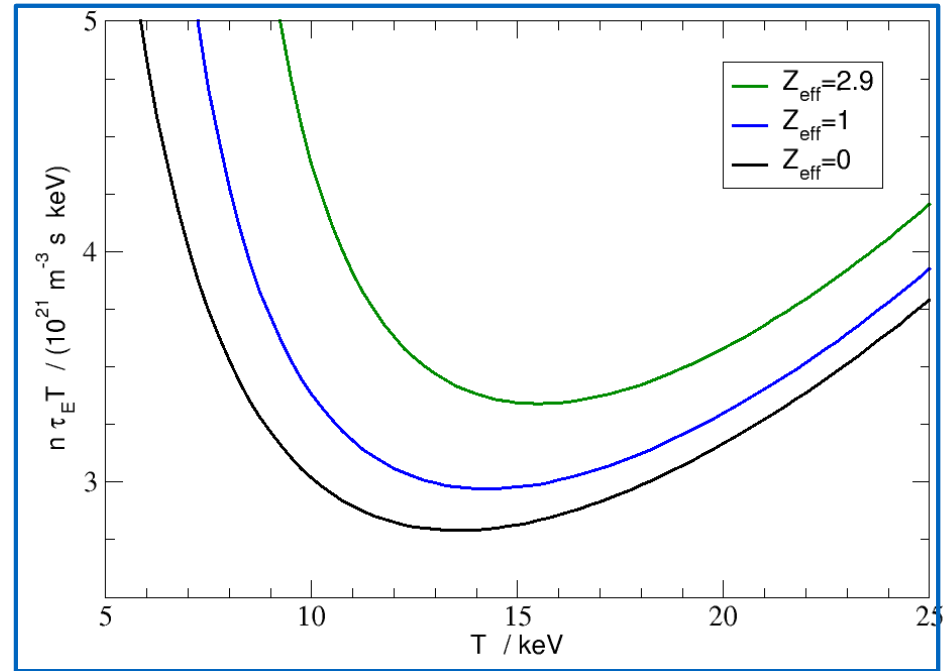
$$P_{\text{Fusion}} = \frac{1}{4} n^2 \langle \sigma v \rangle E_{\alpha}$$

- loss due to radiation: $P_{\text{Bremsstrahlung}} = c_1 n_e^2 Z_{\text{eff}} \sqrt{T}$

- loss due to transport: $P_{\text{Transport}} = \frac{3nkT}{\tau_E}$

- (diffusion, turbulence)

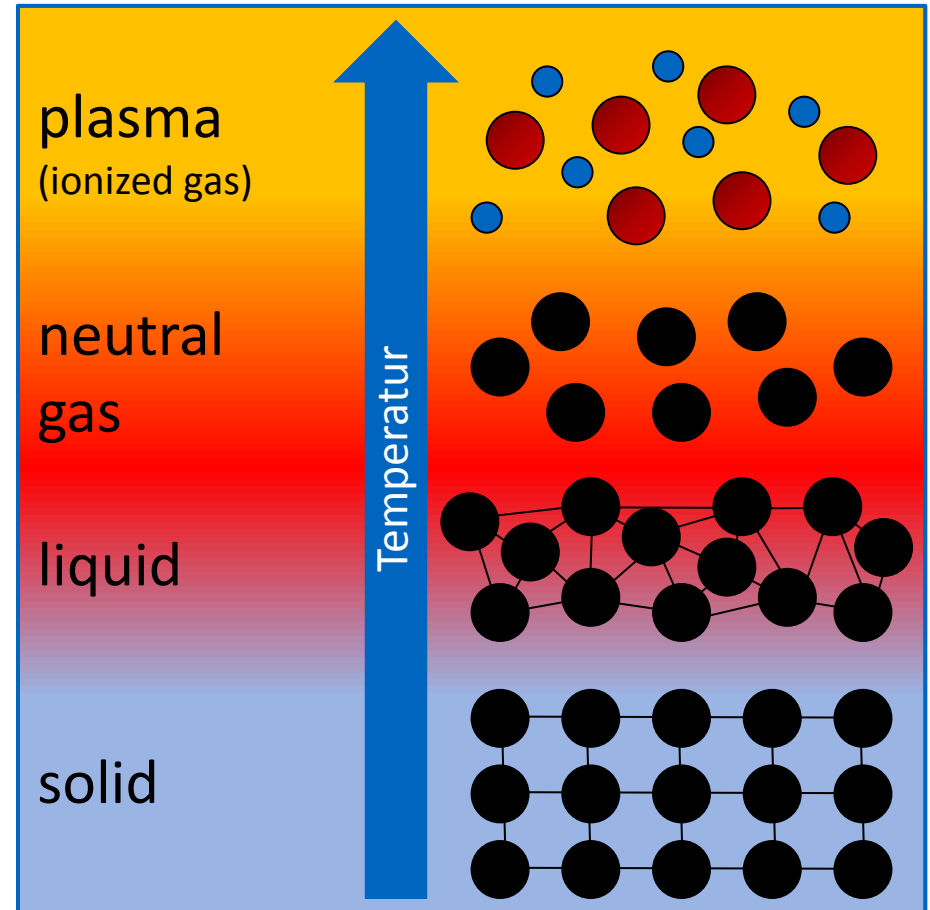
$$\rightarrow n \tau_E T = \frac{12kT^2}{\langle \sigma v \rangle E_{\alpha} - 4c_1 Z_{\text{eff}} \sqrt{T}}$$



source: R. Kleiber 2014

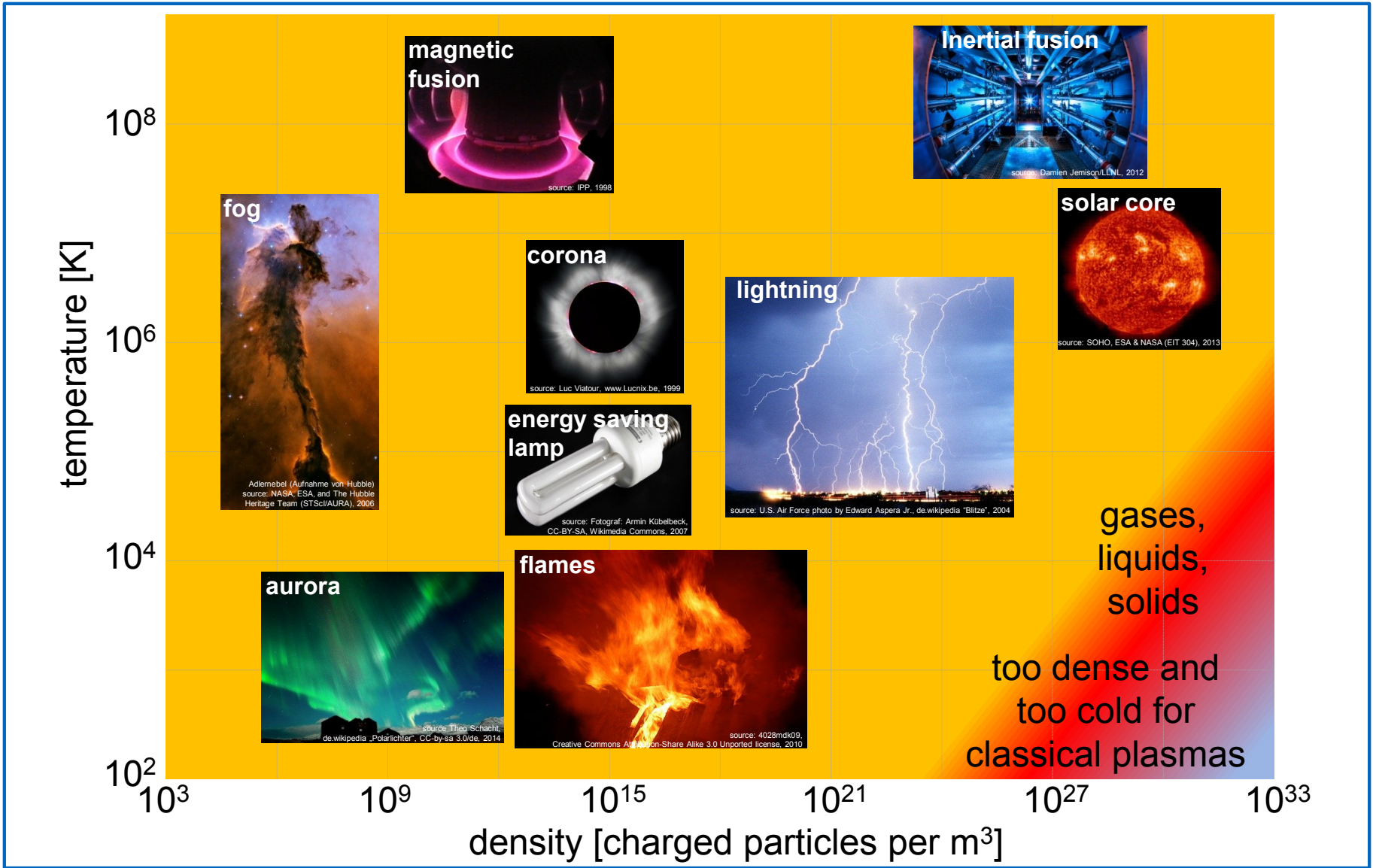
properties of a plasma

- neutral und charged particles
- free electric charge
 - ➔ electric conductivity
- interaction by Coulomb force
 - ➔ collective effects, shielding
- local concentration of charges
 - ➔ oscillations and waves
- no thermodynamical phase transition

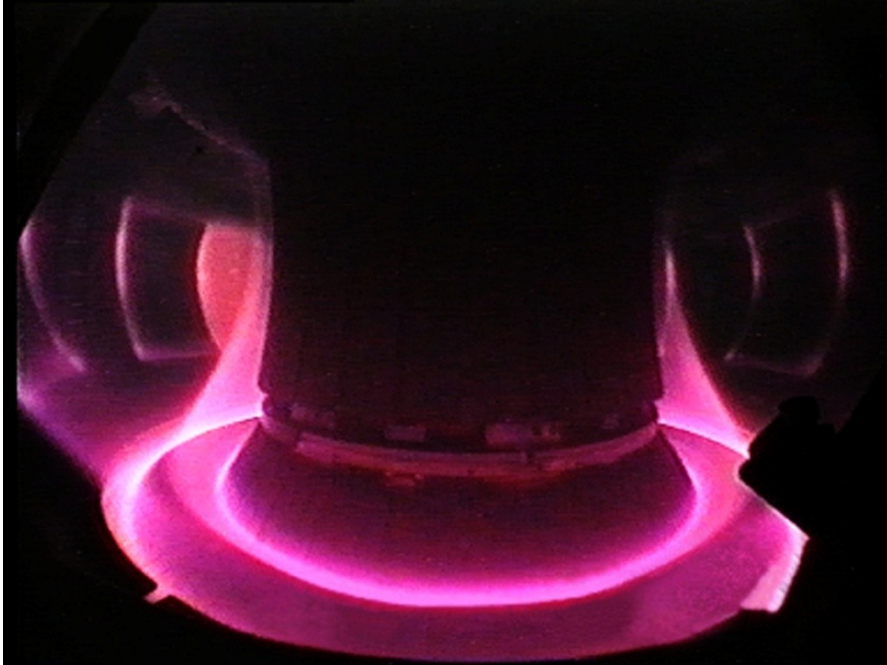


source: A. Kleiber 2014

A vast fraction of matter in the universe is in the plasma state.



source: A. Kleiber 2014



source: IPP 1998




source: A. Ullmann 2012

$T = 150 \text{ million } ^\circ\text{C}$
 $n = 10^{20} \text{ m}^{-3}$



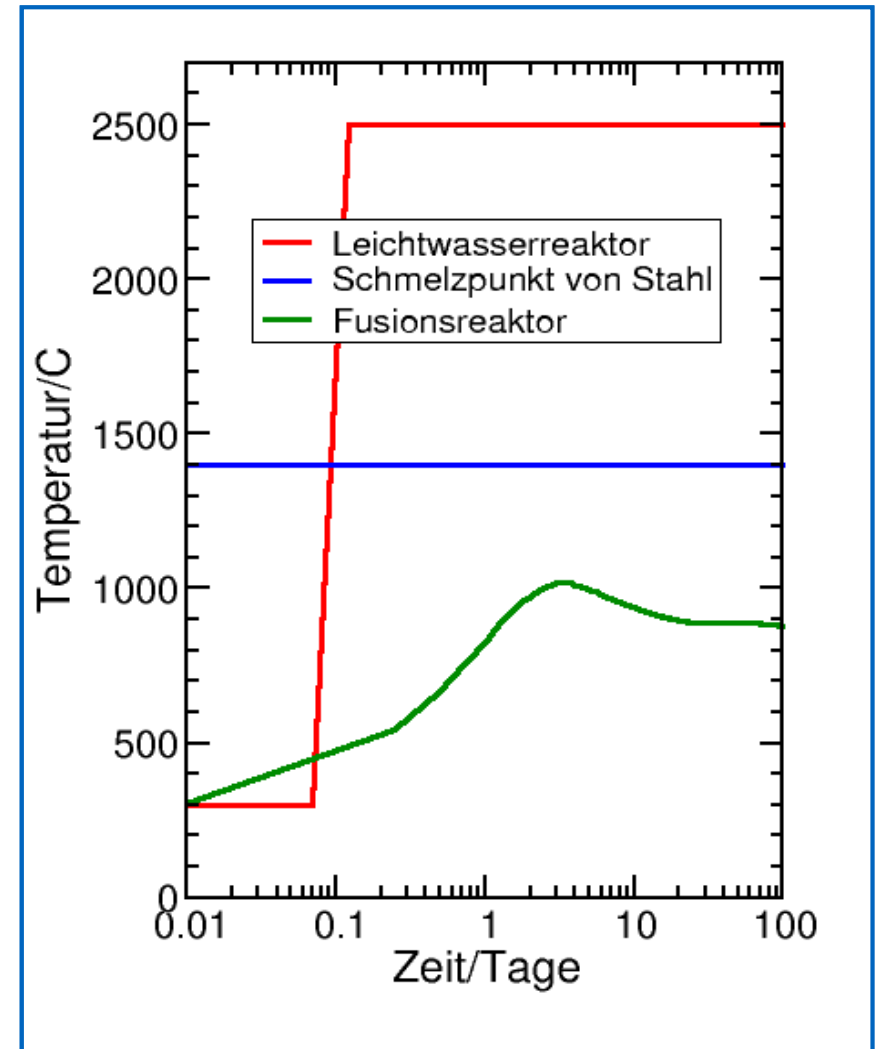
150 CU

- no problems with criticality (no chain reaction)
- small energy reservoir in the plasma
 - fusion reactor contains only fuel for some seconds (fission reactor contains fuel for more than one year)
- low energy density in the plasma
 - high temperature, but low density
 - plasma cannot breach the vessel
- tritium (half-life period $T_{1/2} = 12,3$ years)
 - about 1 g fuel inside a reactor and about 2 kg bound in the wall
 - low radiotoxicity
 - radiation does not penetrate the skin
- neutrons from fusion reaction (14,1 MeV)
 - adequate shielding necessary
 - activation of structure components (waste  materials research)

study of possible accident scenarios

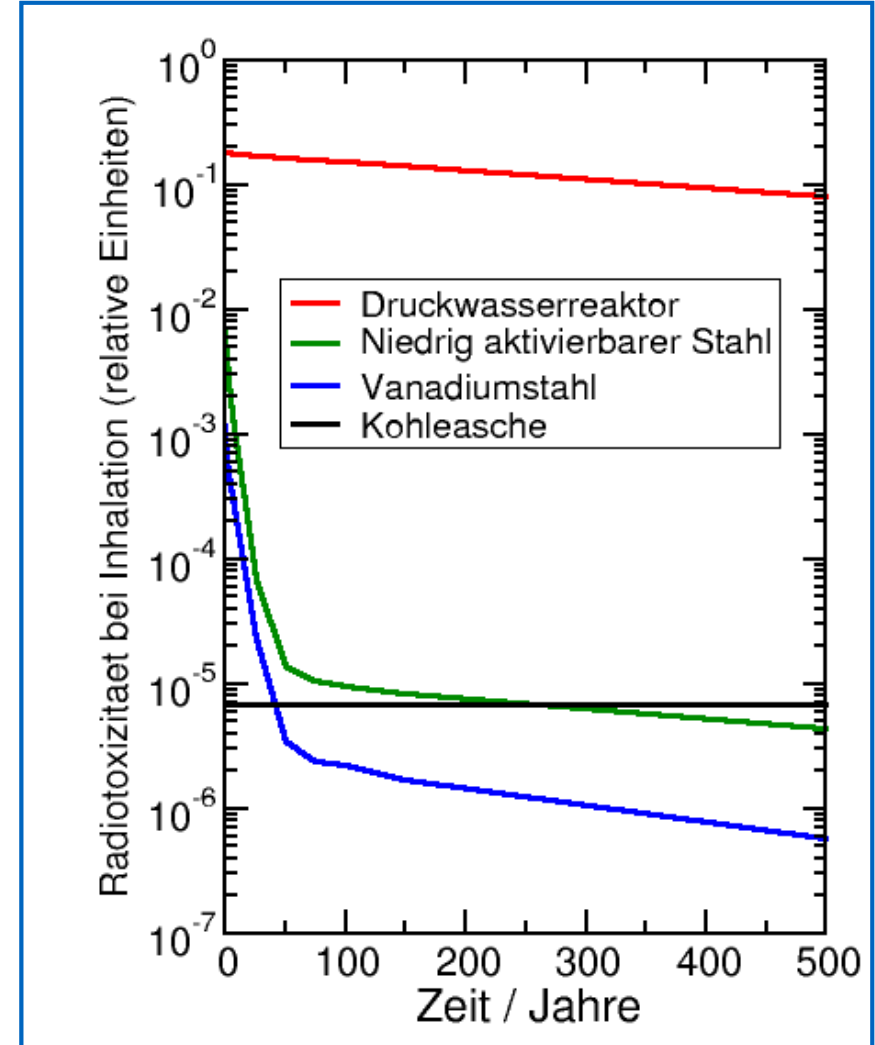
e.g. loss of cooling liquid:

- fusion process will stop immediately
- further warming only through decay heat of activated components
- temperature never reaches melting point of steel
- no meltdown of the reactor
- longer response time for possible intervention



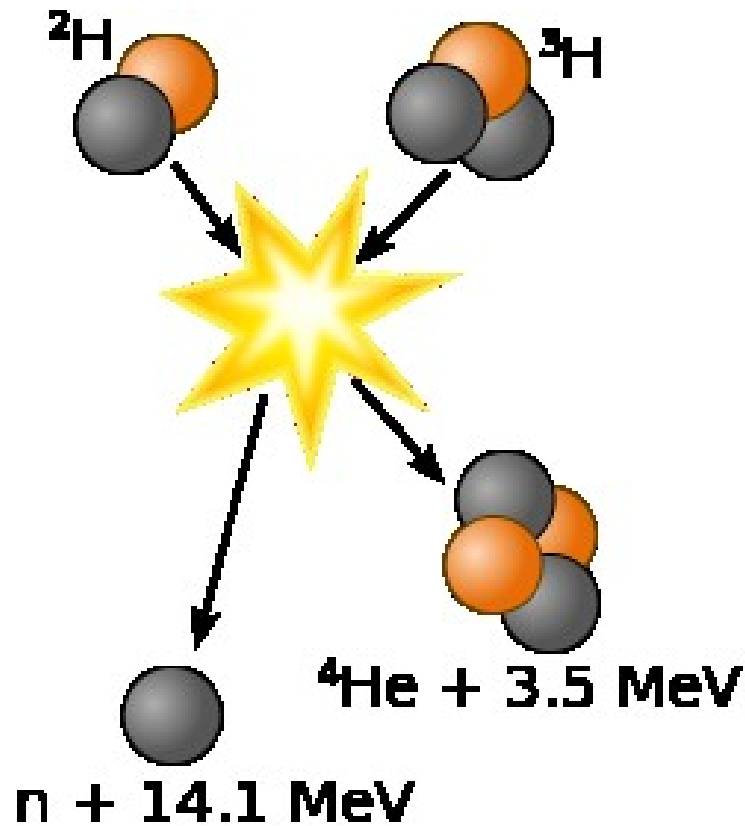
source: EFDA Power Plant Conceptual Study (PPCS), 2005, R. Kleiber, M. Borchardt 2014.

- radioactive waste results from steel activated by neutrons
 - optimisation by improved materials
(materials research)
 - no long-living isotopes produced (e.g. U, Pu, Th)
 - after 100 years 90 % of the waste is recyclable
 - after 500 years the remaining 10 % of the waste is comparable with coal ash
- ➔ no final storage necessary

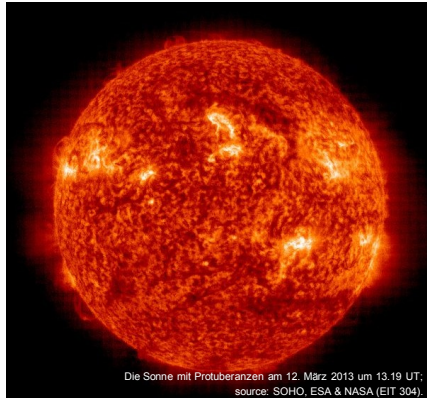


source: EFDA Safety and Environmental Impact of Fusion (SEIF), 2001, R. Kleiber, M. Borchardt 2014

BASIC CONCEPTS OF NUCLEAR FUSION

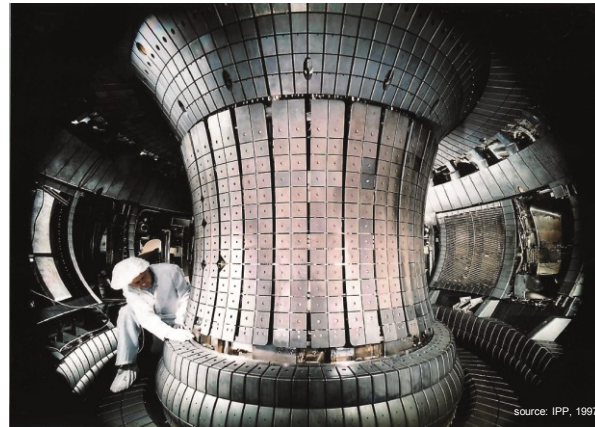


source: de.wikipedia "Kernfusion", Wykis, 2007

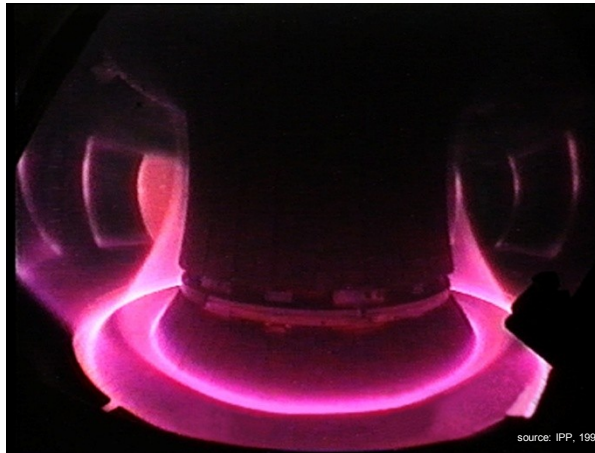


Die Sonne mit Protuberanzen am 12. März 2013 um 13:19 UT;
source: SOHO, ESA & NASA (EIT 304)

gravitation
(stars)

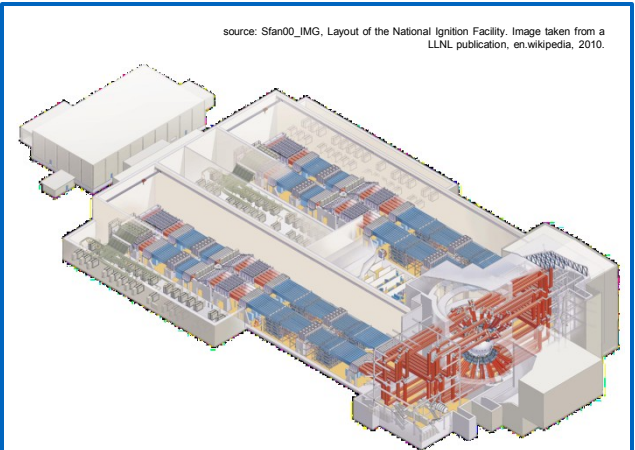


source: IPP, 1997

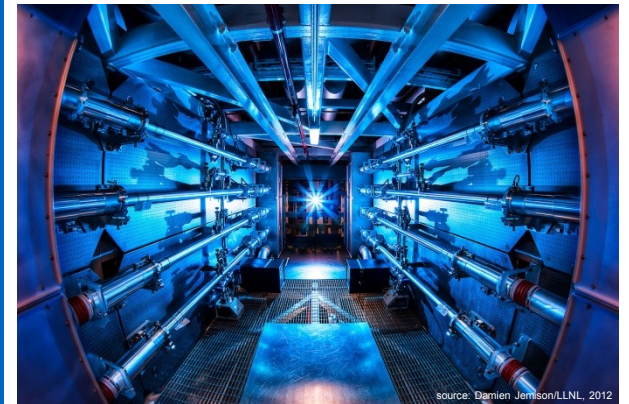


source: IPP, 1998

magnetic confinement

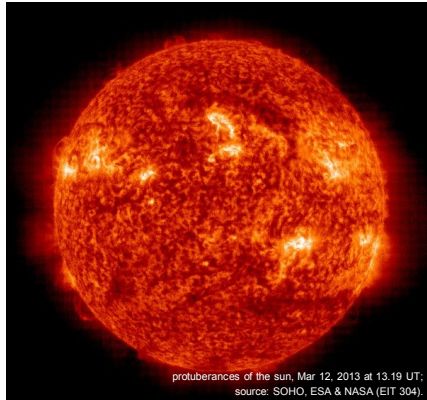


source: Stan00_IMG, Layout of the National Ignition Facility. Image taken from a LLNL publication, en.wikipedia, 2010.



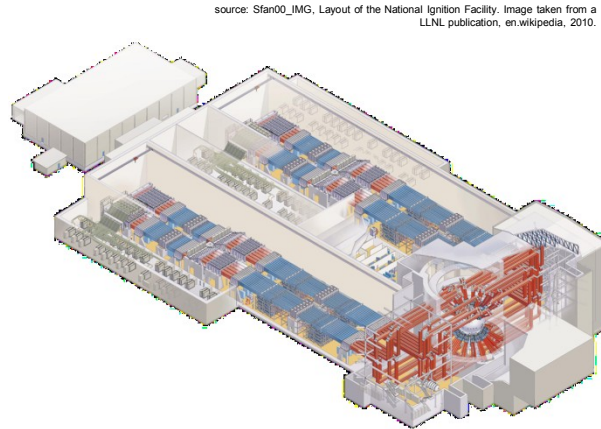
source: Damien Jemison/LLNL, 2012

inertial fusion

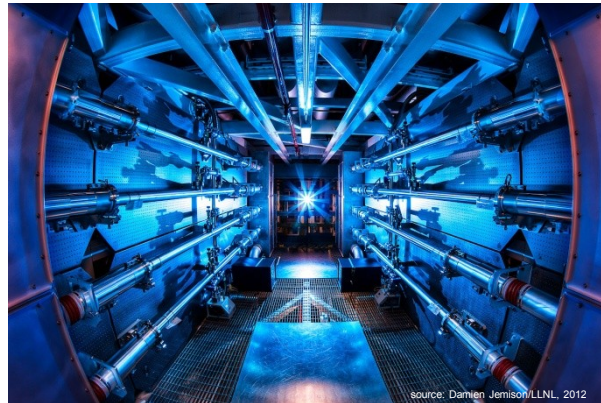


protuberances of the sun, Mar 12, 2013 at 13:19 UT;
source: SOHO, ESA & NASA (EIT 304)

gravitation
(stars)

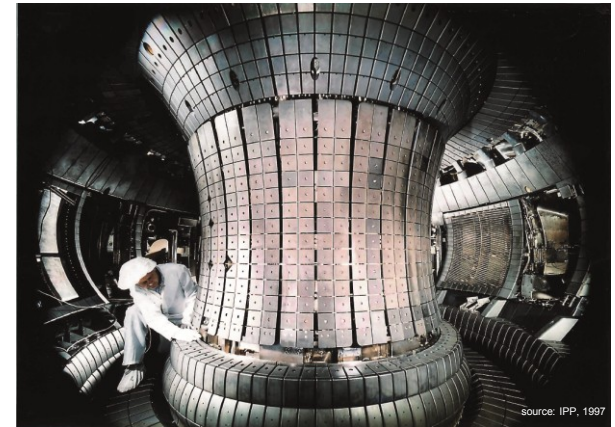


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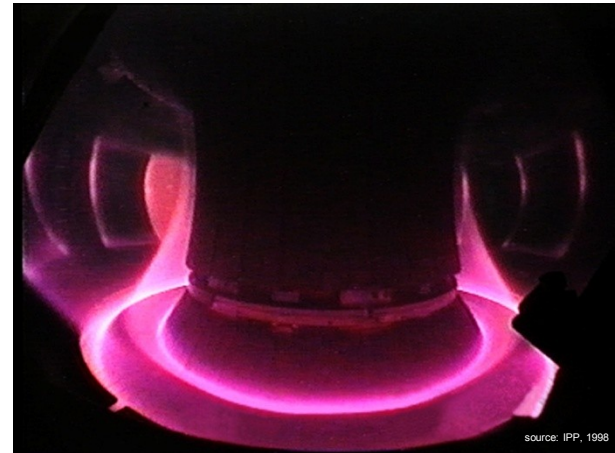


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inertial confinement



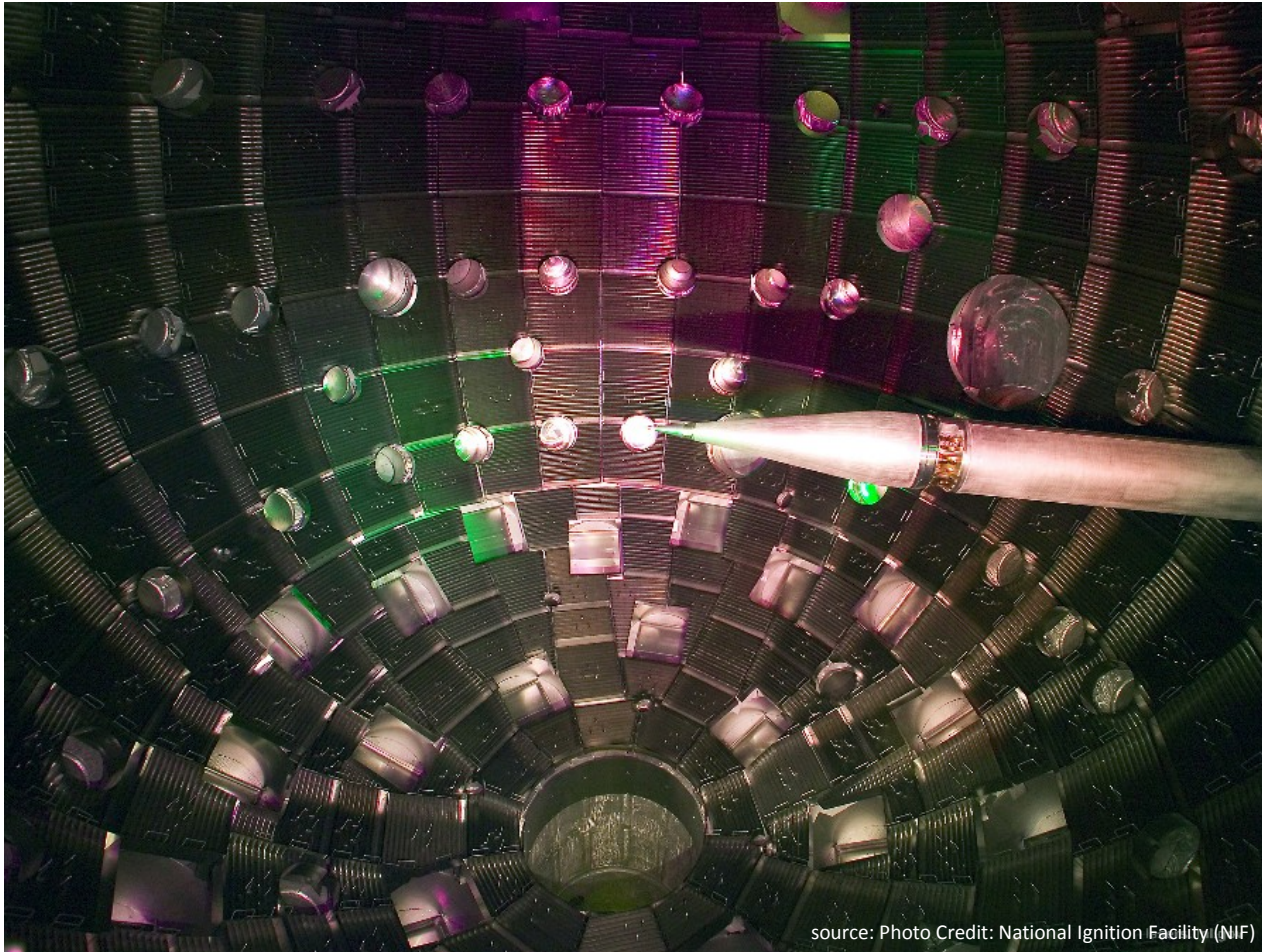
source: IPP, 1997



source: IPP, 1998

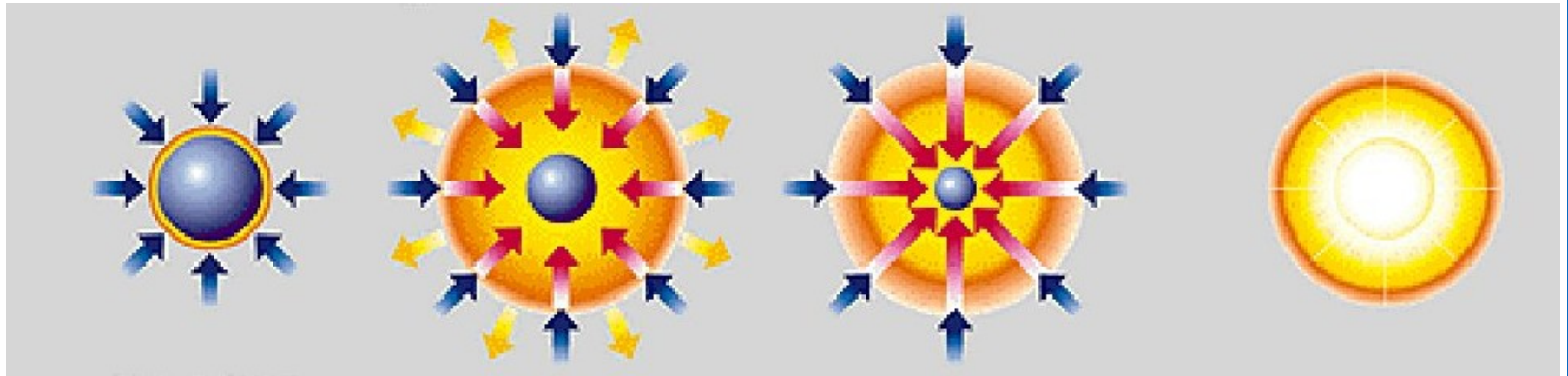
magnetic confinement

INERTIAL FUSION



source: Photo Credit: National Ignition Facility (NIF)

- laser energy
- blow-off
- energy transported inwards



1. heating
by laser beams,
plasma layer on
the surface

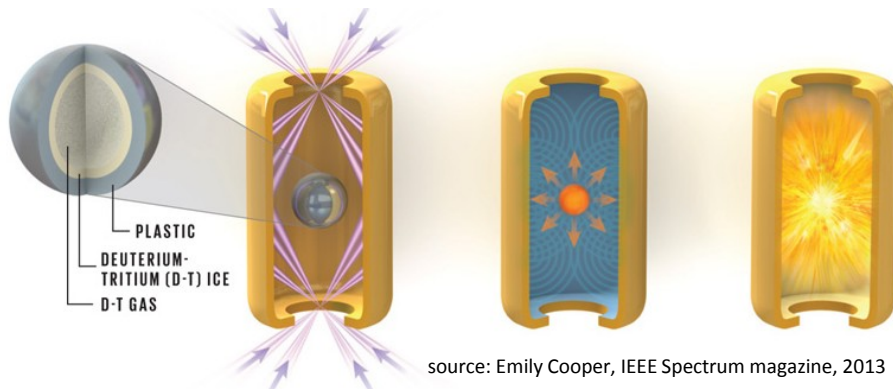
2. compression
by ablation of
surface material

3. ignition
at the end of the laser
pulse the centre
reaches 100 times the
mass density of lead
and 100 million °C

4. burning
rapid spreading of
thermonuclear
burn
→ energy gain

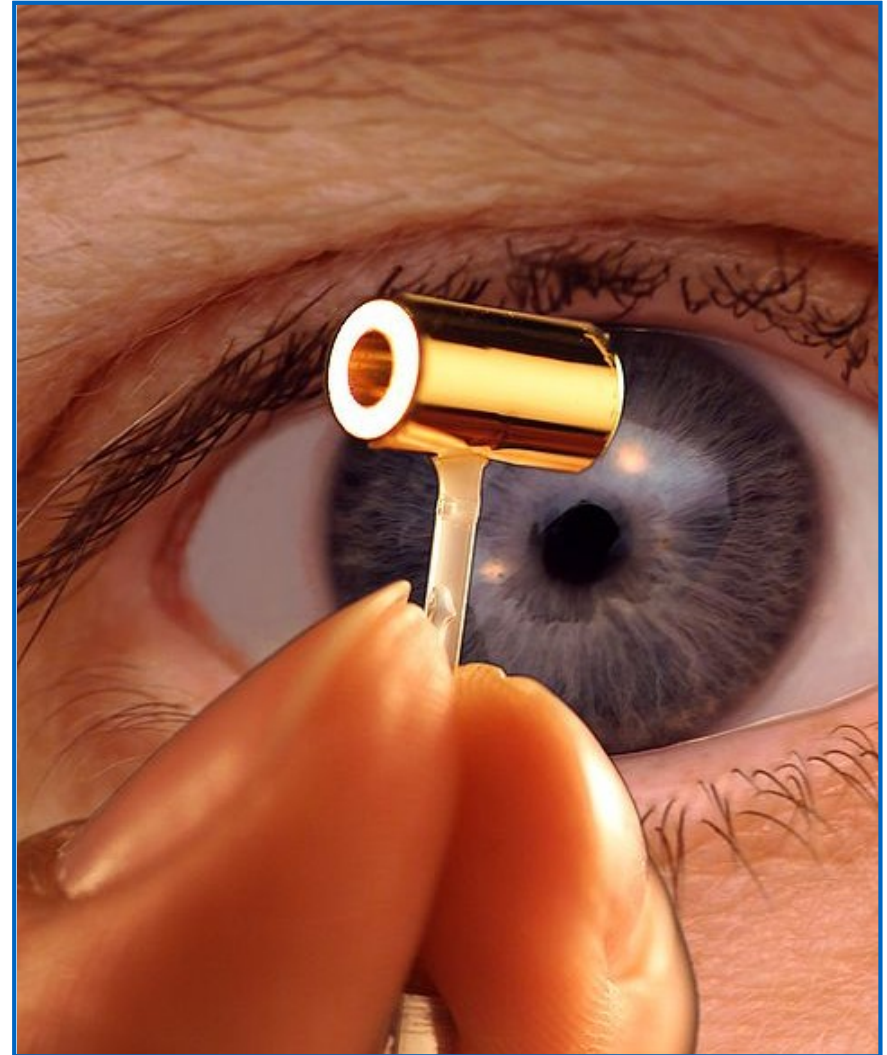
source: NIF, A. Puls 2005.

metallic hohlraum of high atomic number (e.g. gold) contains a microcapsule with fuel (target)



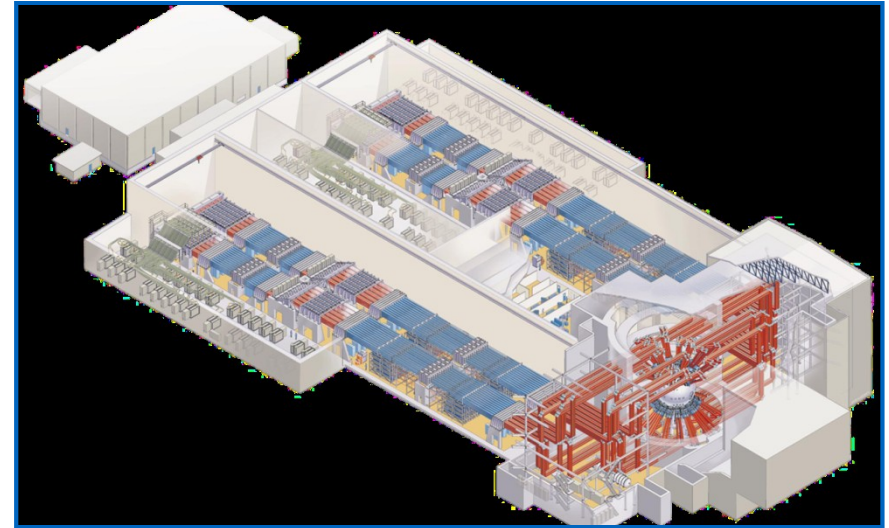
- lasers heat the inside
- thermal X-rays are emitted

homogeneous radiation causes a uniform compression of the fuel



source: Croquant, en.wikipedia „inertial fusion“, llnl.gov, 2006

- completed 2009
- 192 lasers
- 500 TW power
- 1.85 MJ of laser energy injected into target chamber
- milestone 2013:
released more energy than absorbed by the fuel
- official mission:
 - defence research
 - basic research
 - inertial fusion

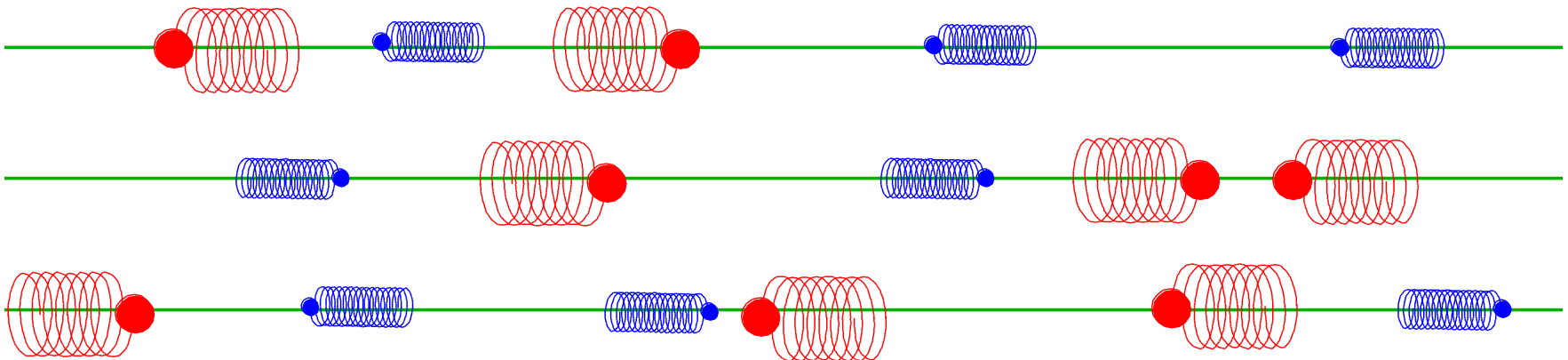


source: Sfan00_IMG, Layout of the National Ignition Facility. Image taken from a LLNL publication, en.wikipedia, 2010



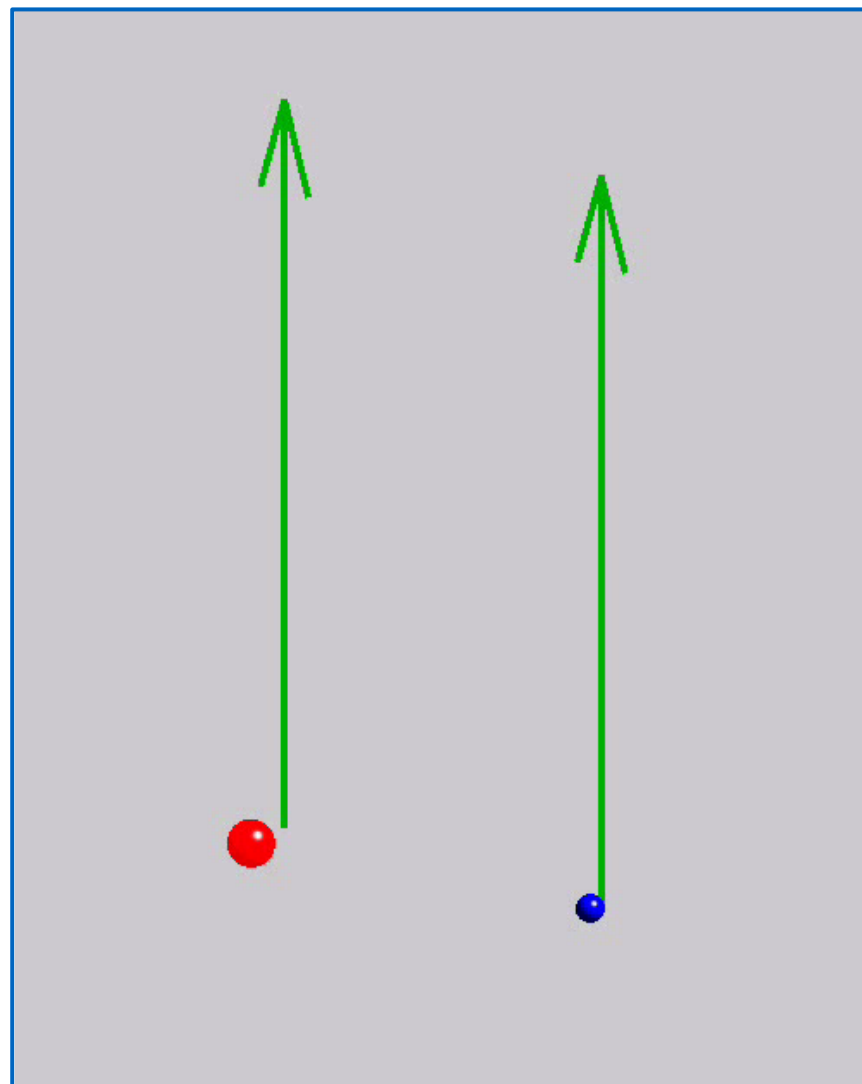
source: Laser Bay 2, National Ignition Facility, Lawrence Livermore National Laboratory, 2007

MAGNETIC CONFINEMENT



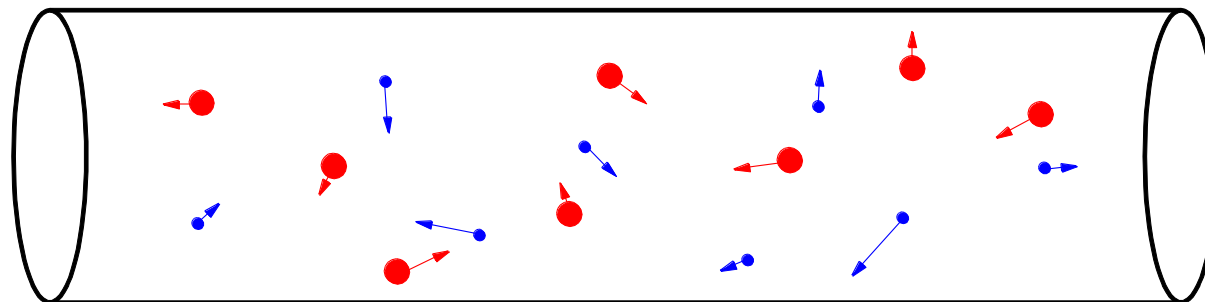
source: R. Kleiber 2012

- a hot plasma cannot be confined by material walls
 - ➔ use magnetic field (Lorentz force)
- magnetic field in fusion devices ca. 2.5 - 6 Tesla (earth magnetic field $\approx 50 \mu\text{T}$)
- gyration radius:
 - ions: 2 mm
 - electrons: 45 μm
- gyration frequencies:
 - ions: 90 MHz
 - electrons: 170 GHz



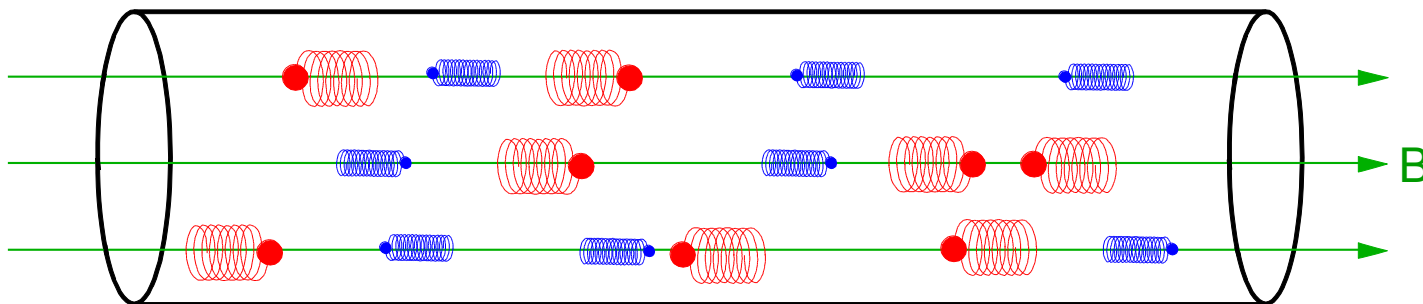
source: R. Kleiber 2012

without magnetic field



- ion
- electron

with magnetic field



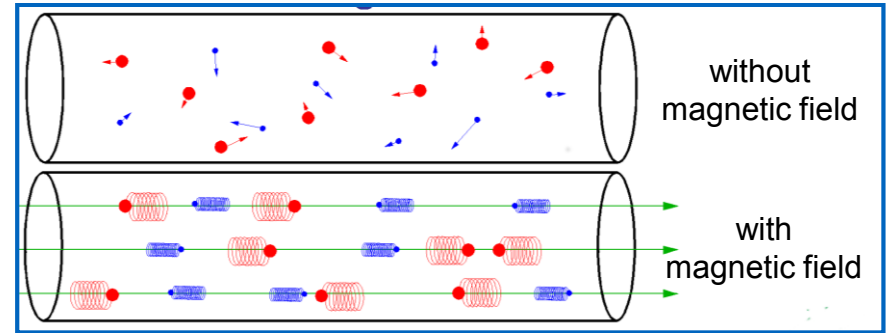
source: R. Kleiber 2012

Lorentz Force in Magnetic Field

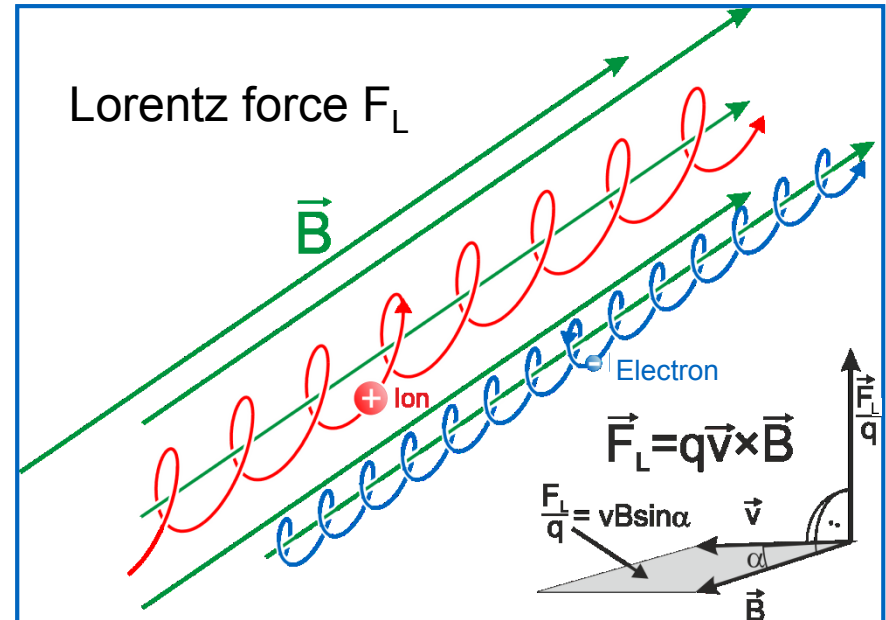
Hot plasmas cannot be confined by conventional material walls.

Force on electric charges in a magnetic field:

- motion perpendicular to field is modified
- particles forced on helical trajectories
- transport properties are changed
- fusion devices: ca. 2.5 - 6 Tesla (corresponds to ca. 10^5 -times earth magnetic field strength)

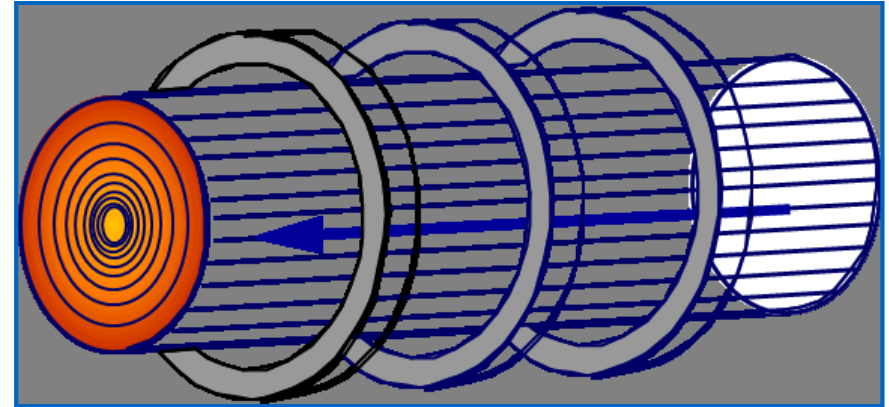


source: R. Kleiber 2012

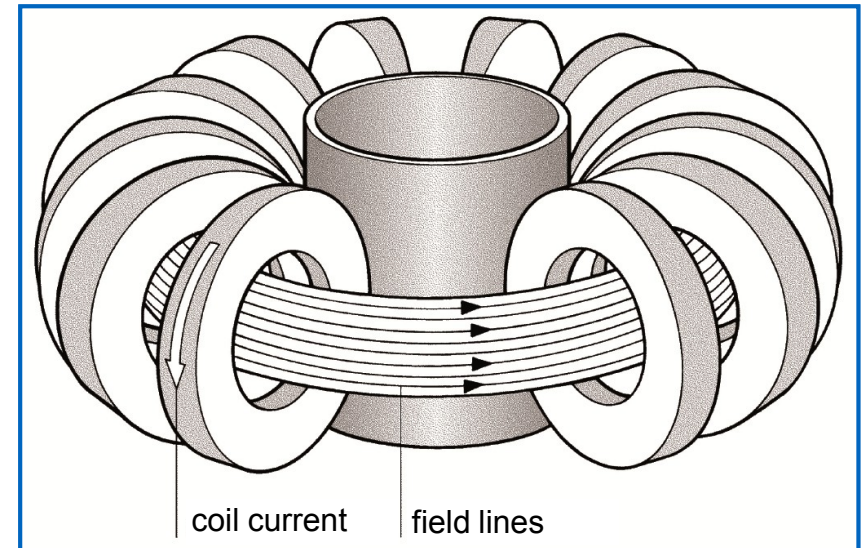


source: A. Kleiber 2014

- cylindrical homogenous magnetic field
 - confinement perpendicular to axis
 - **but: particles lost at the ends**
- closing the field toroidally
 - quasi-endless configuration
 - **but: particle losses due to drift motion (reason: inhomogenous field and curvature)**



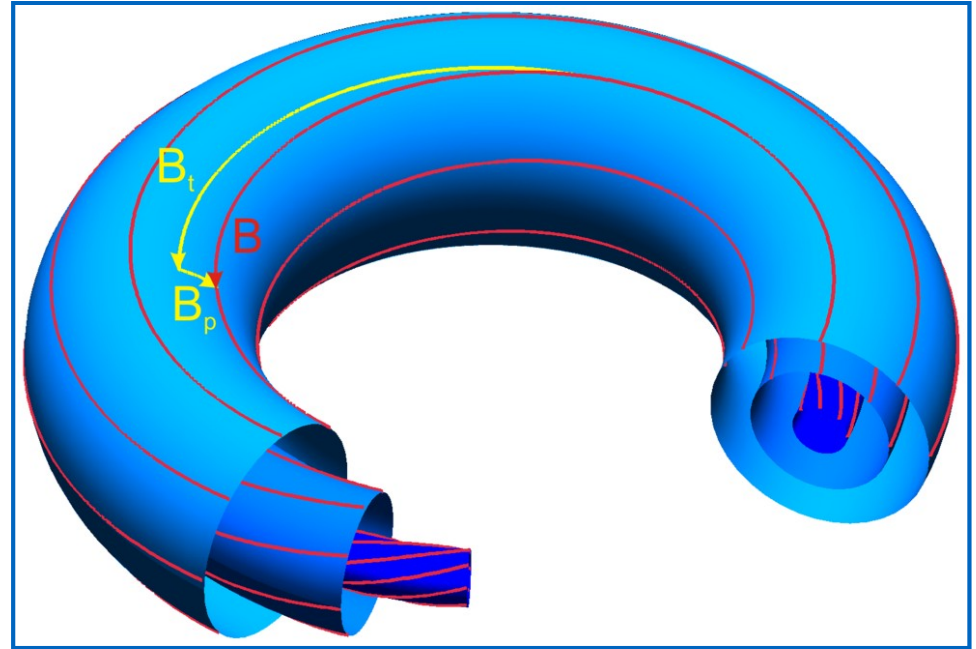
source: IPP



source: IPP 2002

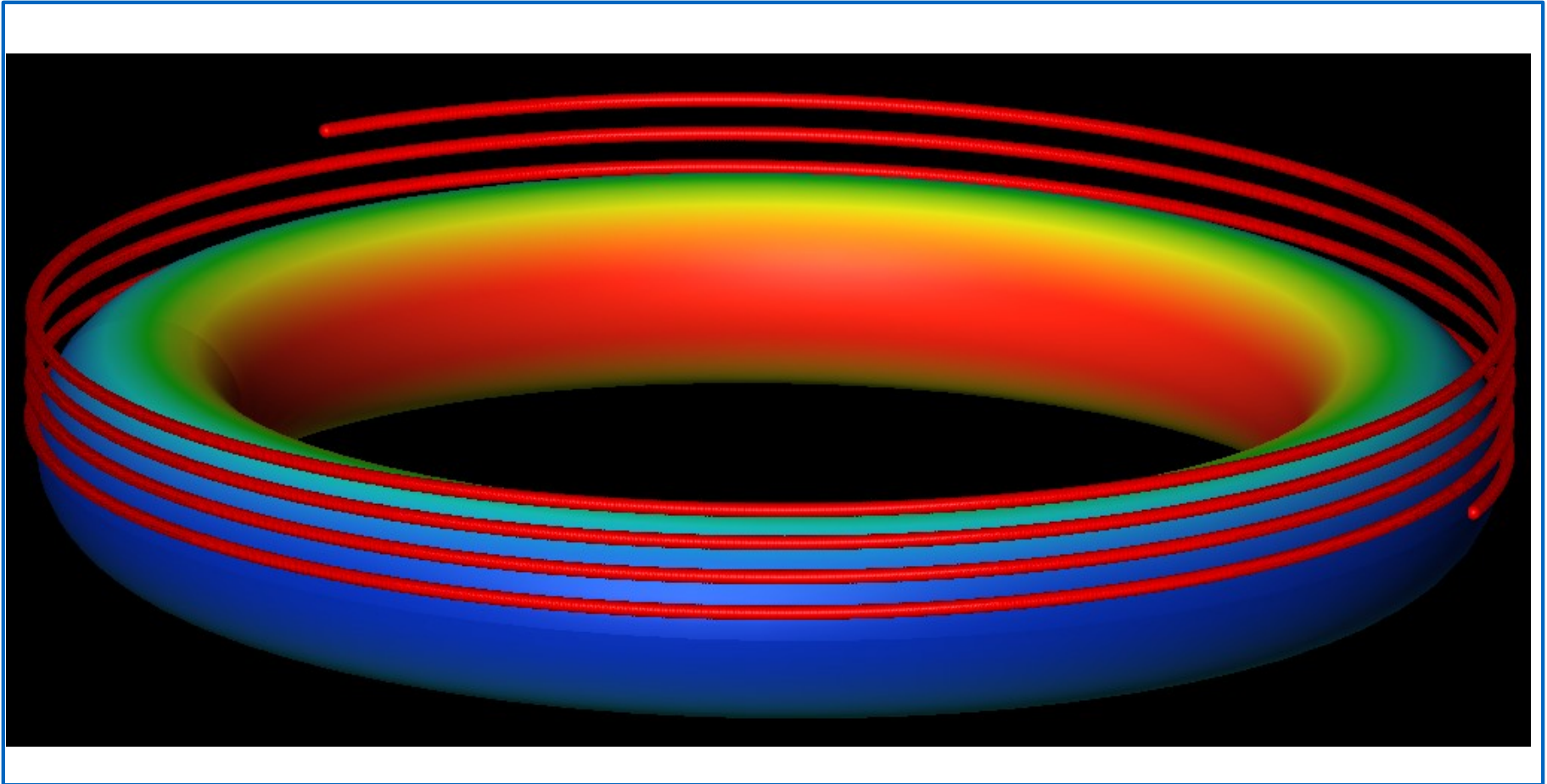
Twisted magnetic fields lead to confinement of particles.

- two field components
 - B_t toroidal field
 - B_p poloidal fieldgive rotational transform
- creation of magnetic flux surfaces
- rotational transform varies over radius



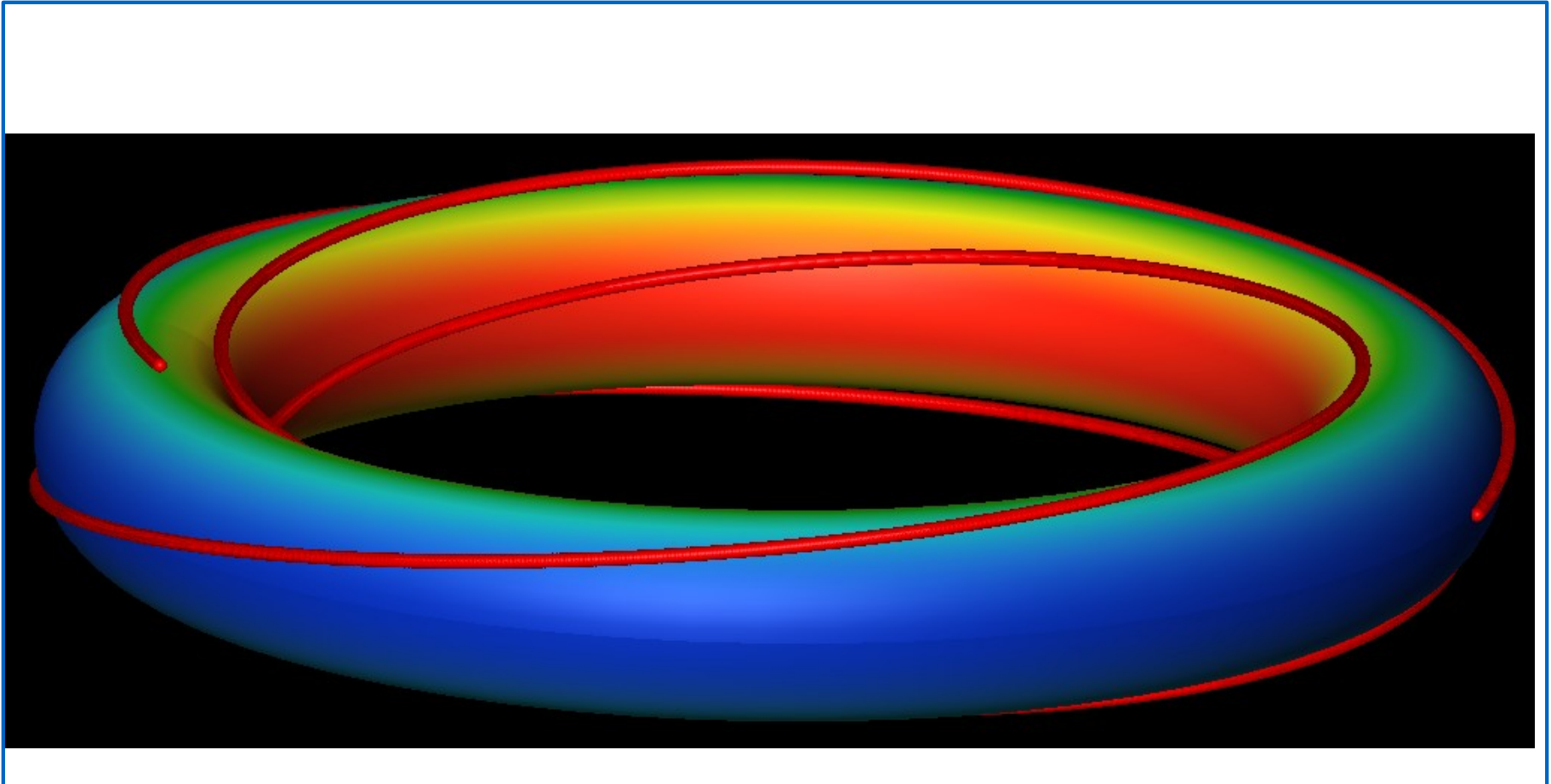
source: R. Kleiber, A. Pulss 2008

two possible concepts to generate rotational transform:
Tokamak & Stellarator



source: M. Borchardt 2014

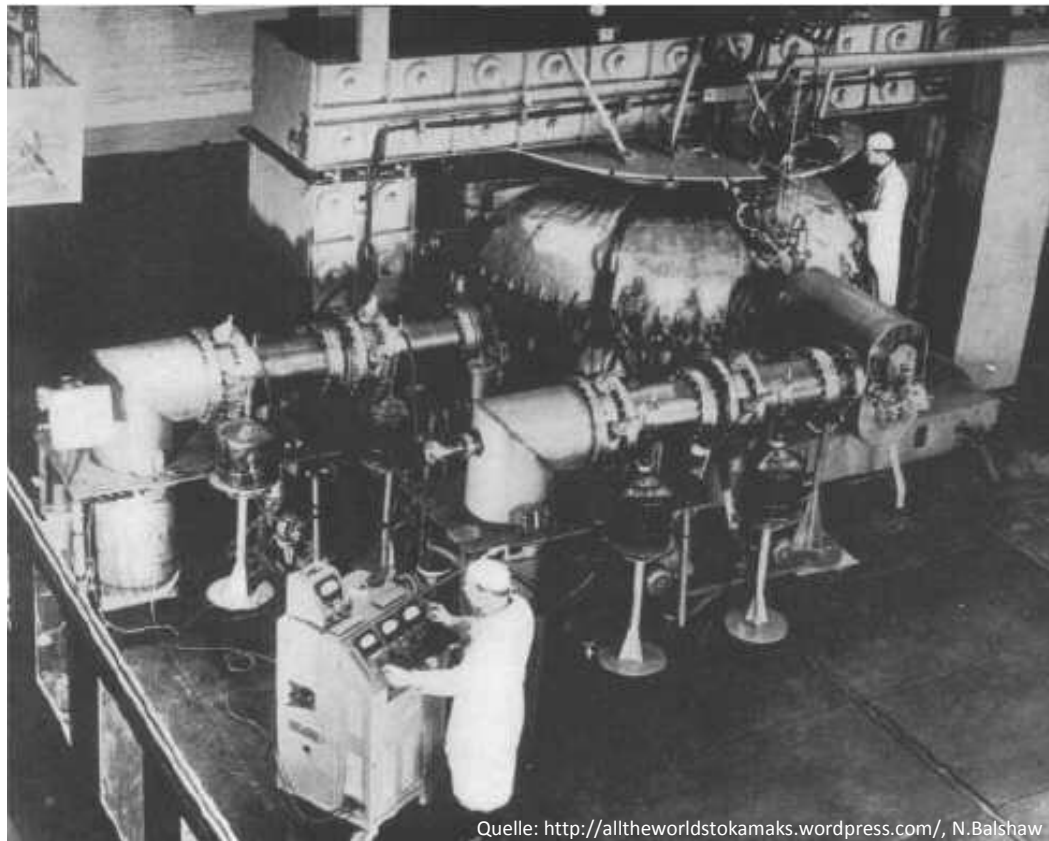
Plasma particles leave the torus.



source: M. Borchardt 2014

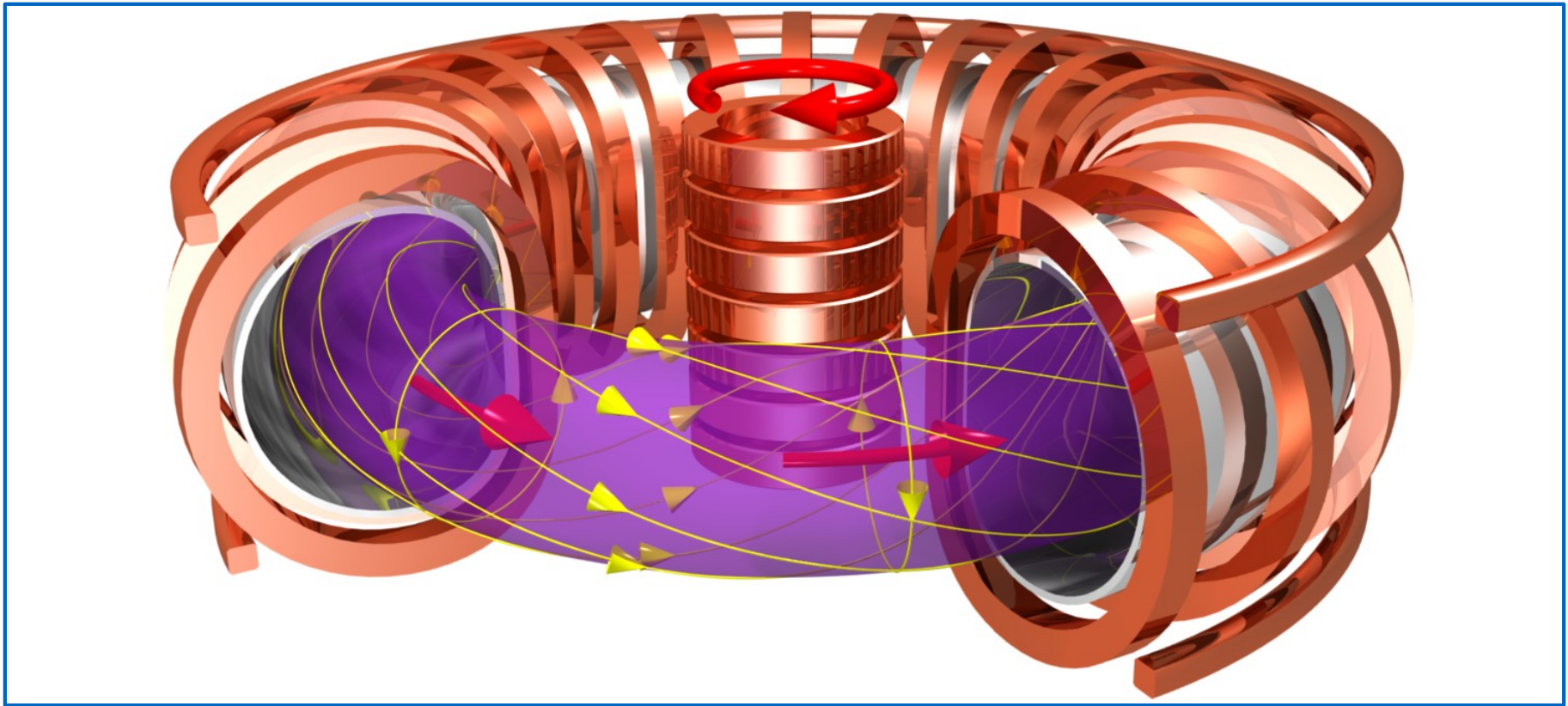
Plasma particles are confined to the torus.

THE TOKAMAK



Quelle: <http://alltheworldstokamaks.wordpress.com/>, N.Balshaw

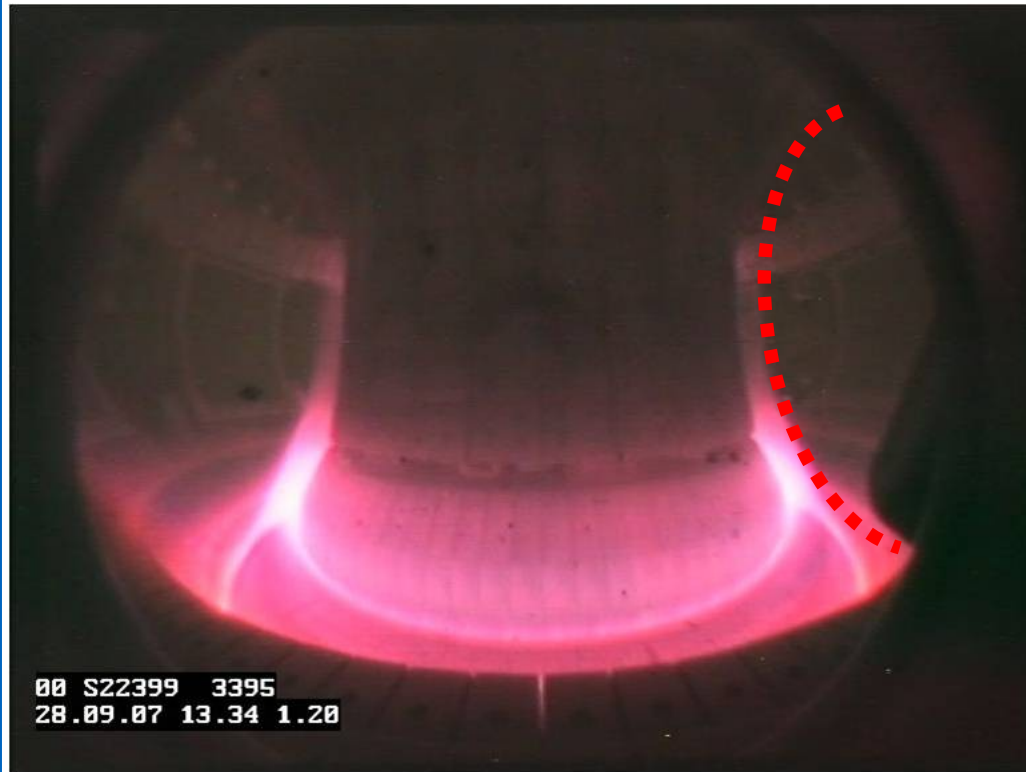
T3-Tokamak, Kurchatov-Institute Moscow, USSR



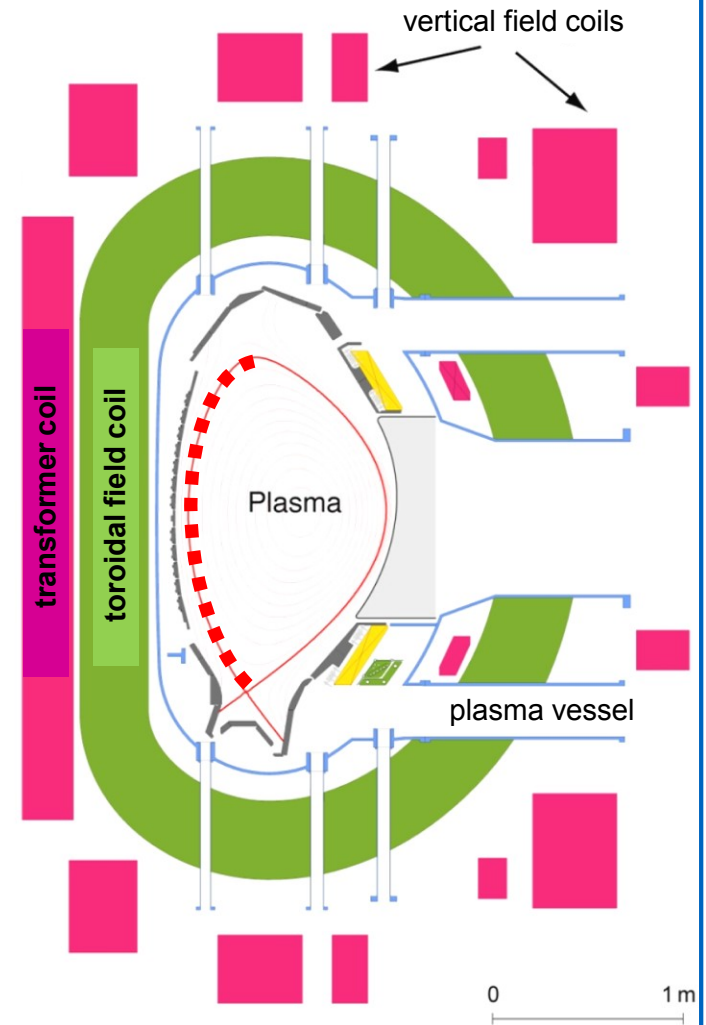
source: Christian Brandt, IPP 2011

developed around 1950 by Artsimovich and Sacharov in Moscow:

- + intrinsic heating
- + most advanced fusion concept
- no stationary operation (works like a transformer) → current drive
- current quench possible (disruption)

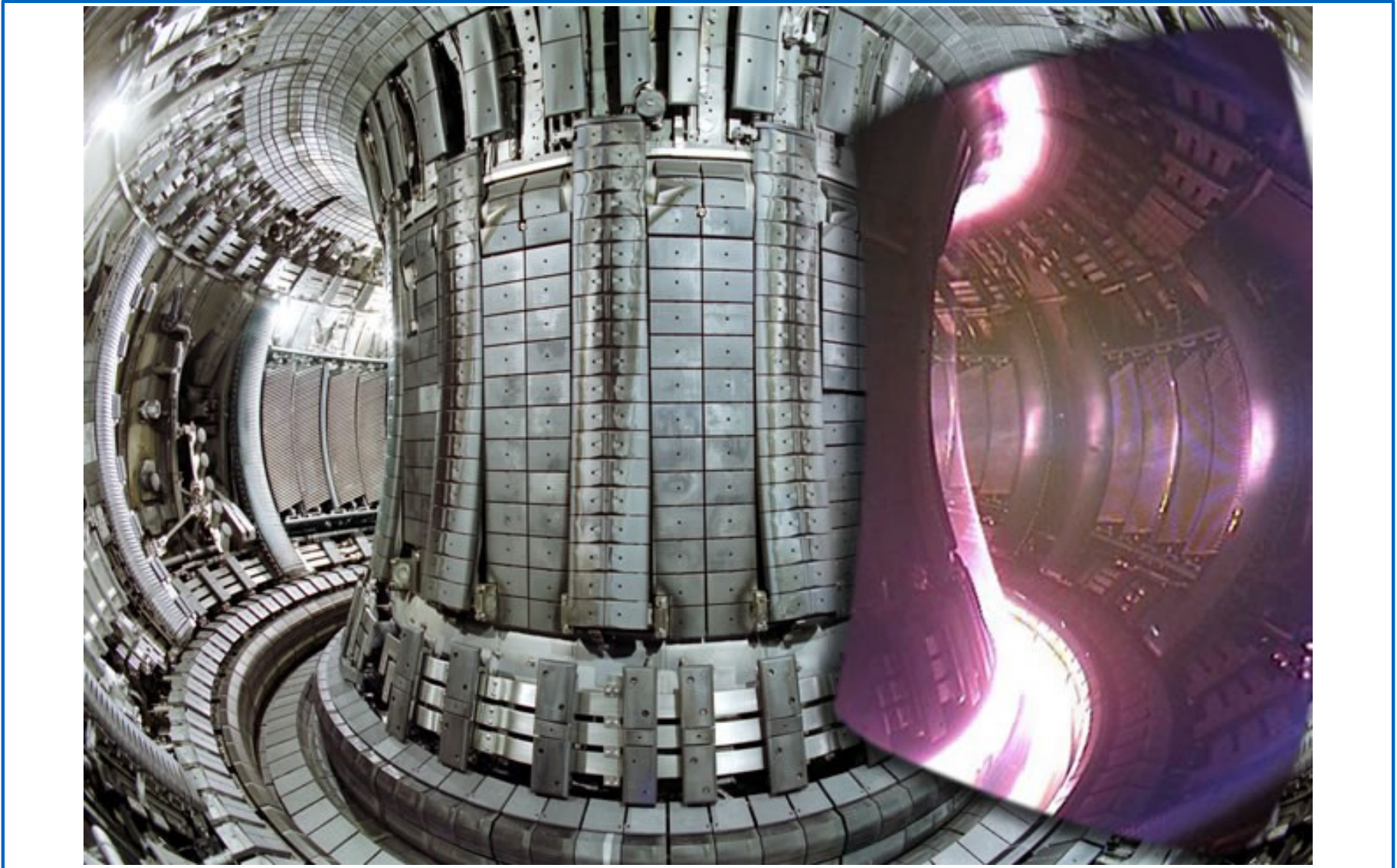


$R = 1.65 \text{ m}$ $a = 0.5 \text{ m}$ $\kappa = 1.6$
 $B_t \leq 3.5 \text{ T}$ $I_p \leq 1.4 \text{ MA}$ $P_H \leq 28 \text{ MW}$

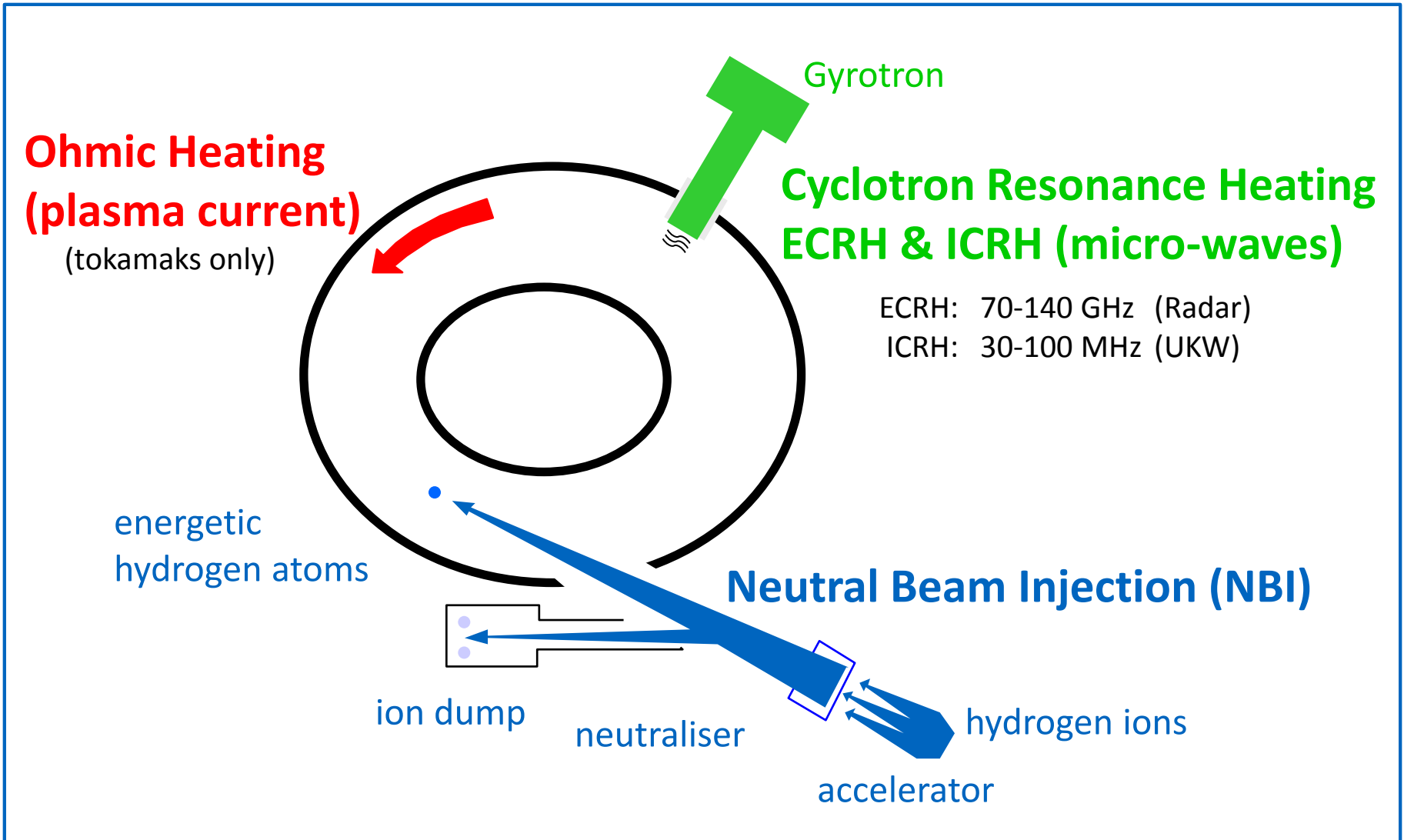


source: IPP 1993

Joint European Torus (JET), Culham, Great Britain



source: EFDA-JET public relations, en.wikipedia "JET"



source: IPP

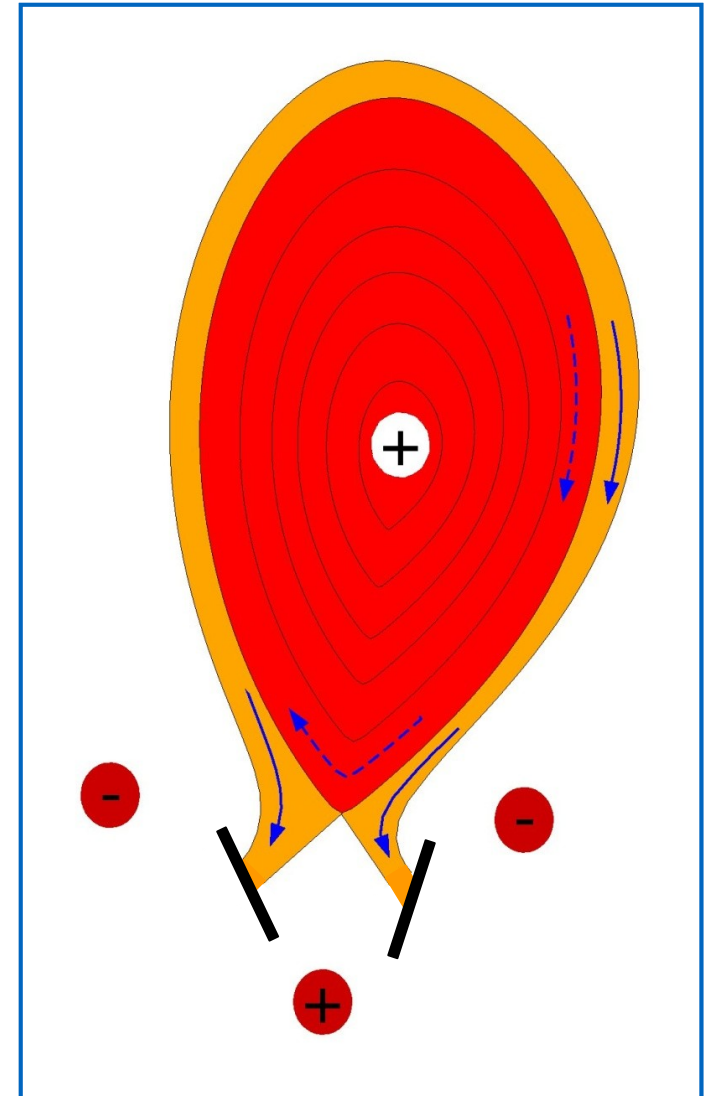
Heating methods in fusion research

- via plasma current (ohmic heating)
applies to tokamaks
- neutral beam injection (NBI):
fast particles dissipate their energy
- cyclotron resonance heating:
accelerates the gyro-motion
 - of ions (ICRH):
30 - 100 MHz (VHF)
 - of electrons (ECRH):
70 - 140 GHz (Radar)



Gyrotron IPP Greifswald, source: M. Borchardt 2014

- plasma confinement with nested and closed magnetic surfaces
- plasma edge needs to be defined magnetically by additional fields forming a last closed surface (separatrix)
- divertor for the first time successfully used at IPP (ASDEX):
 - clean plasma
 - enhanced confinement (H-mode)
- nowadays a standard element of all larger tokamaks and stellarators for well-defined wall contact

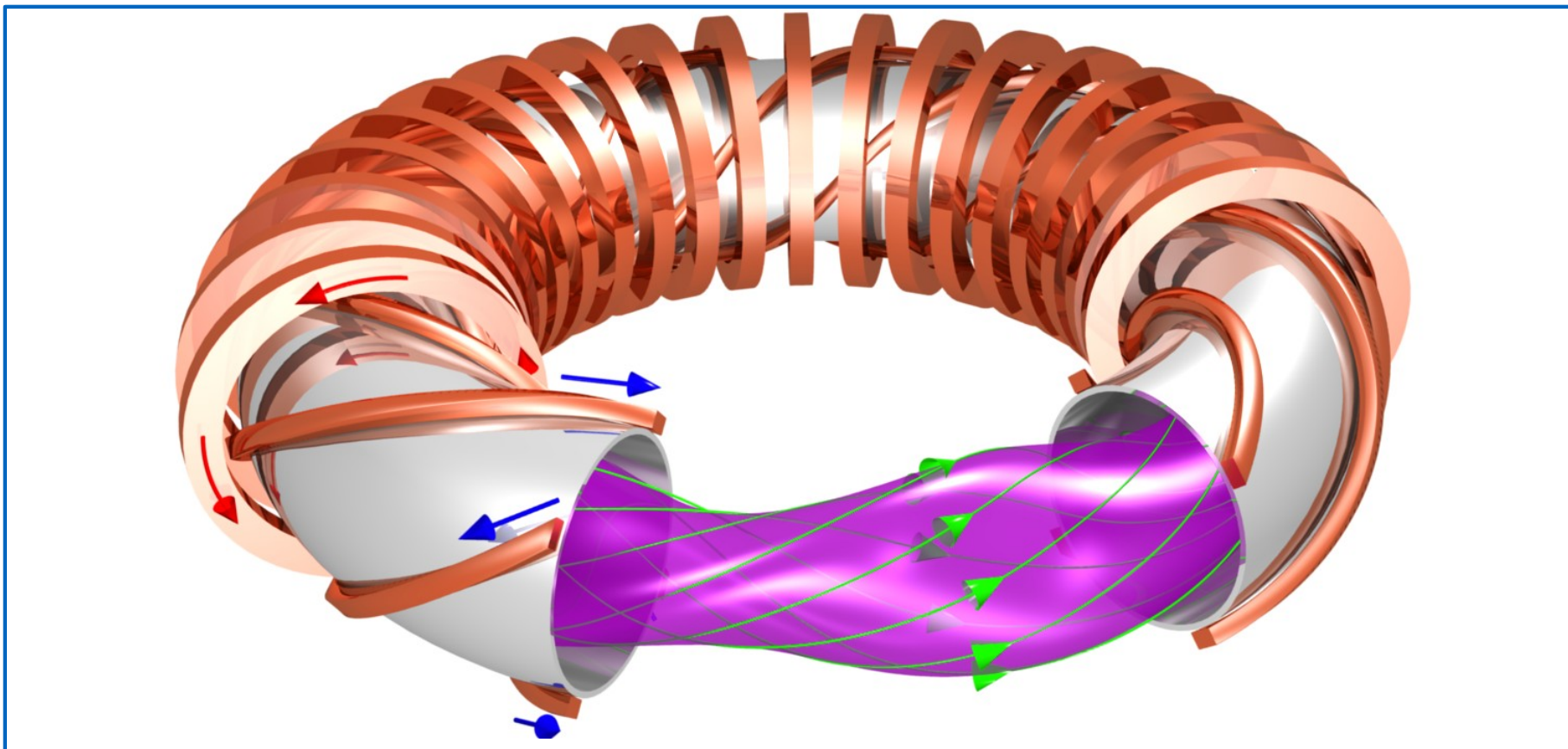


source: IPP

THE STELLARATOR



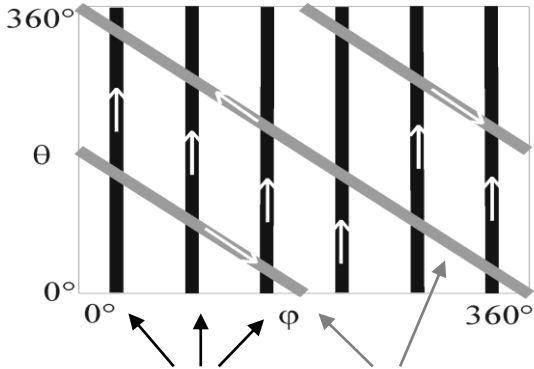
Stellarator A, Princeton Plasma Physics Laboratory (PPPL), USA



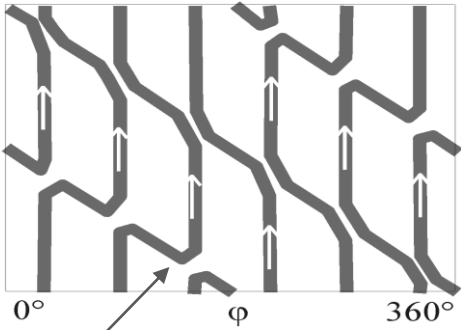
source: Christian Brandt, IPP 2011

- + external currents only
- + good controllability
- + stationary operation
- ⇒ need and potential for optimisation

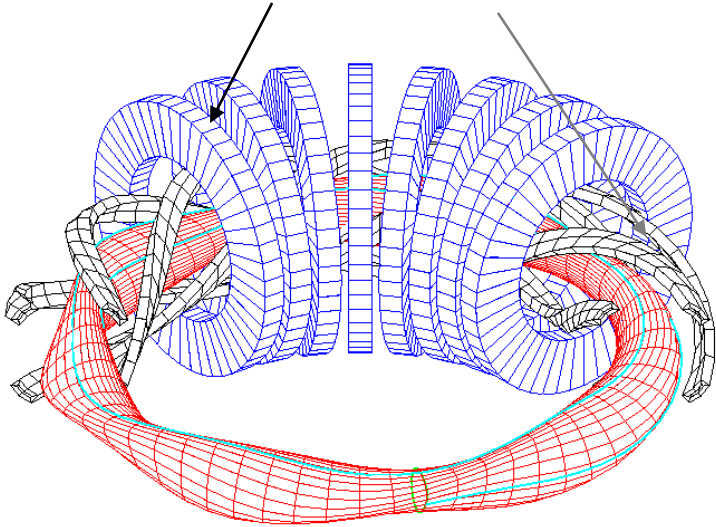
- complex three-dimensional geometry
- reflected particles not confined



toroidal field coils helical field coils



modular coils



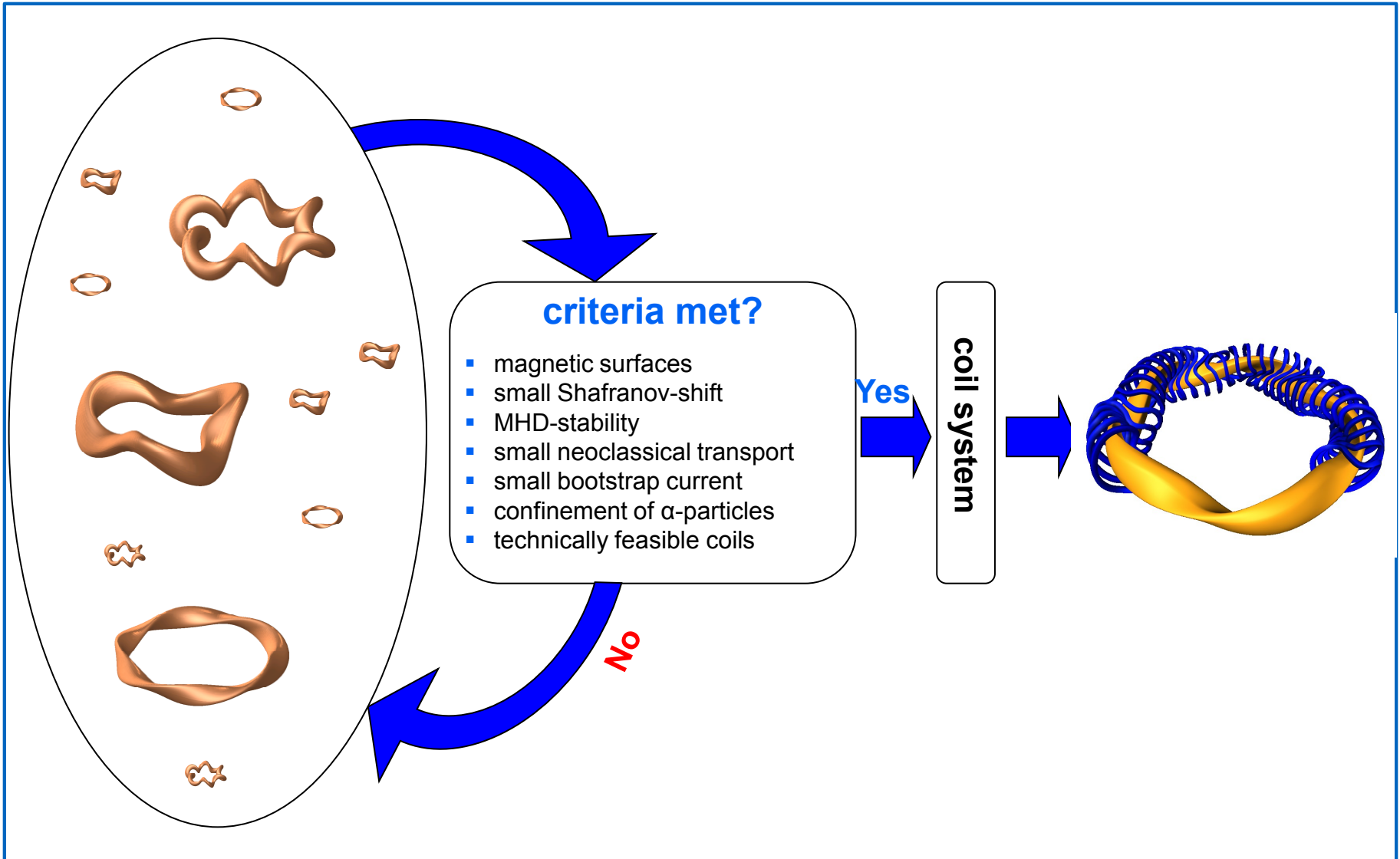
nested

source: IPP

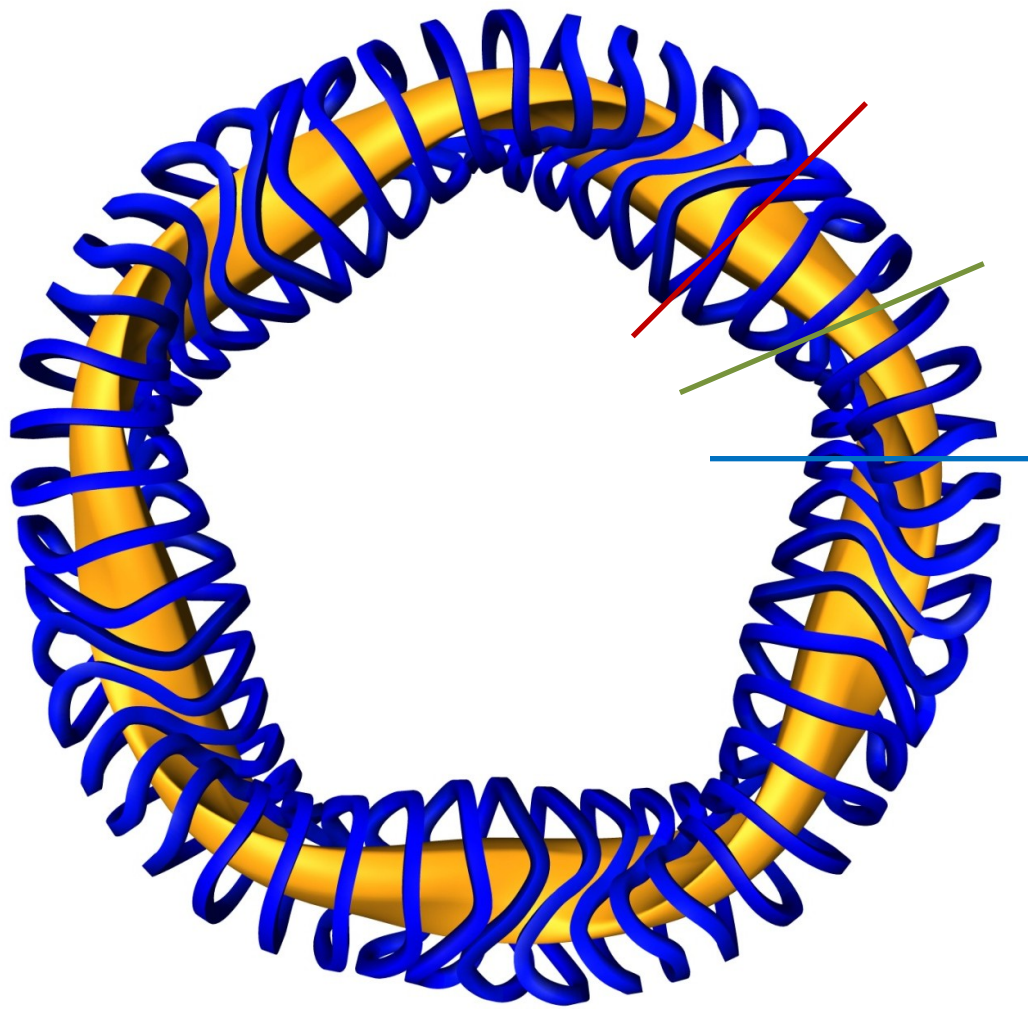
FUSION EXPERIMENT WENDELSTEIN 7-X



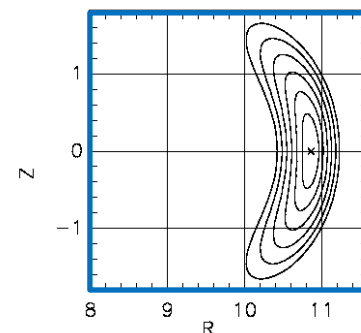
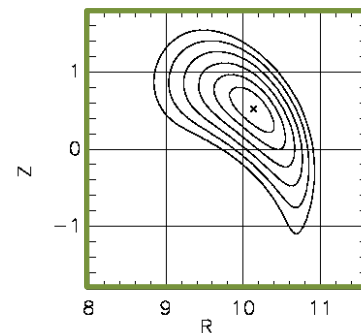
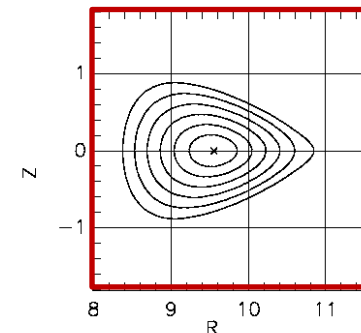
source: B. Kemnitz 2014



source: R. Kleiber, M. Borchardt 2012

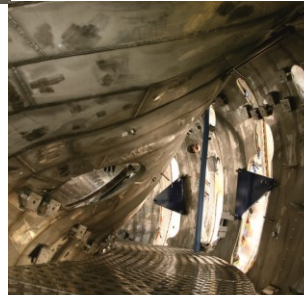
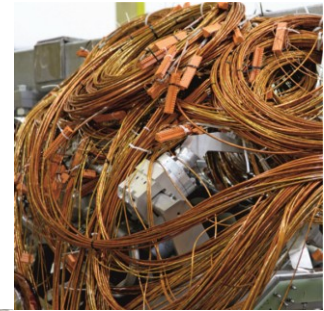
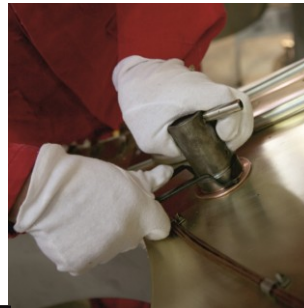
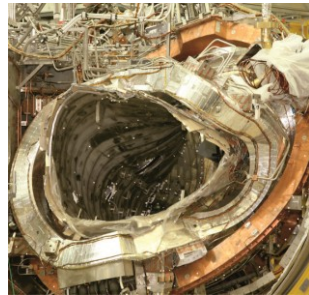


plasma cross section



source: M. Borchardt, R. Kleiber 2012/2014

ASSEMBLY OF WENDELSTEIN 7-X

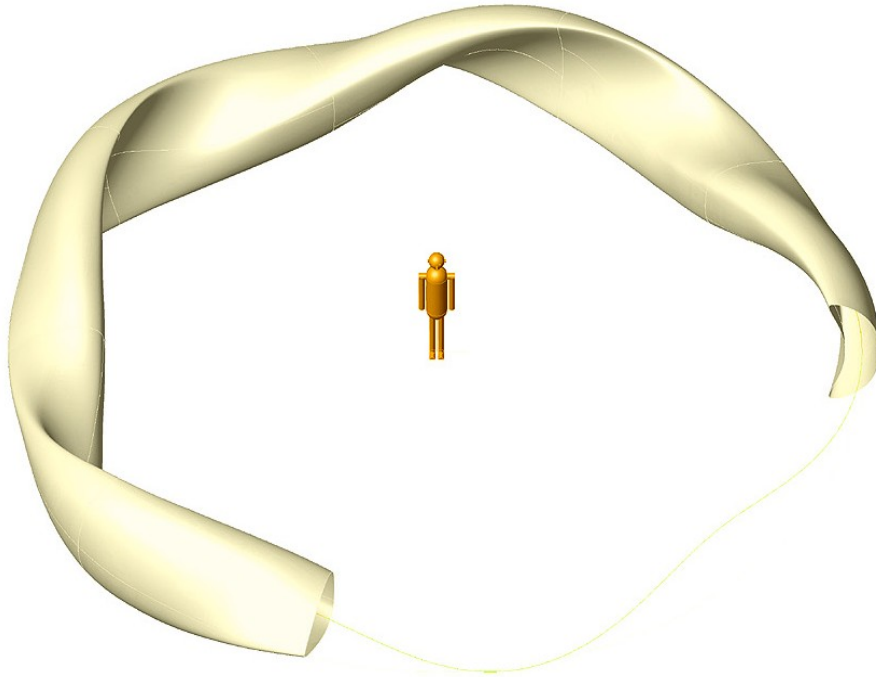


sources: IPP

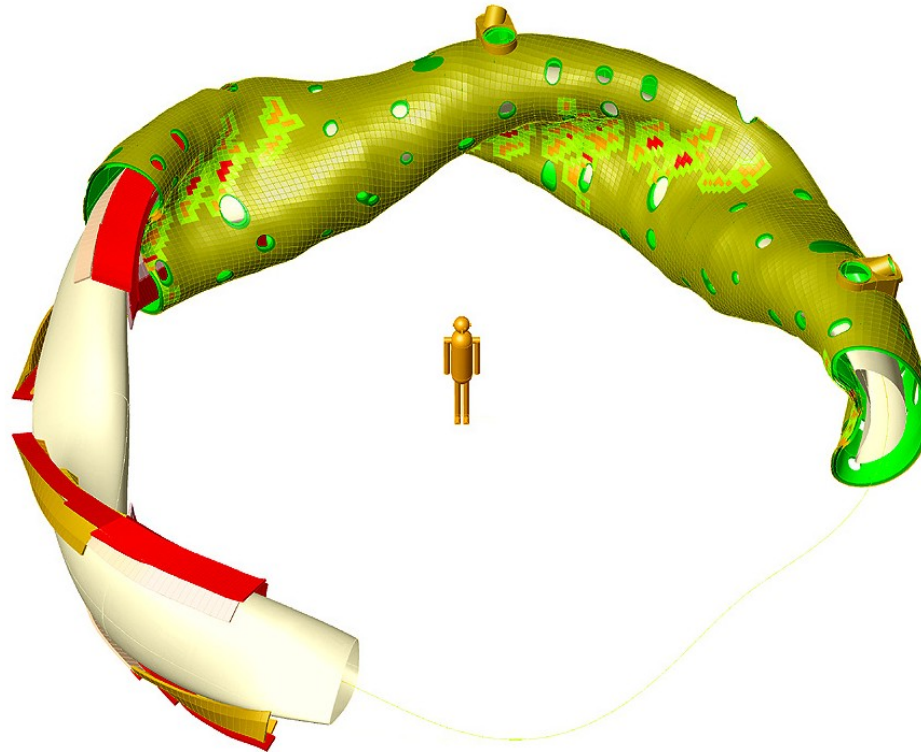
Plasma

R: 5.5 m

$\langle a \rangle$: 0.53 m



source: IPP



**plasma
divertor**

plasma vessel

volume: 110 m³

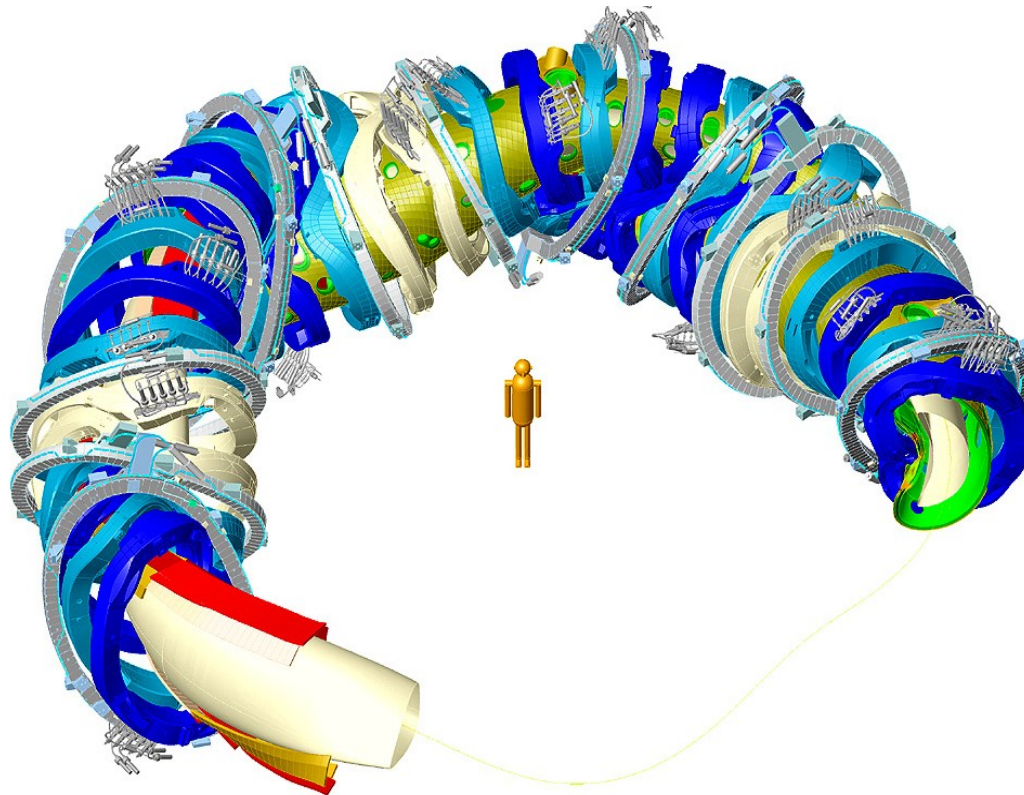
surface: 200 m²

vacuum: < 10⁻⁸ mbar

mass: 35 t

tolerances: < ± 2 mm

source: IPP



plasma
divertor
plasma vessel

coil system

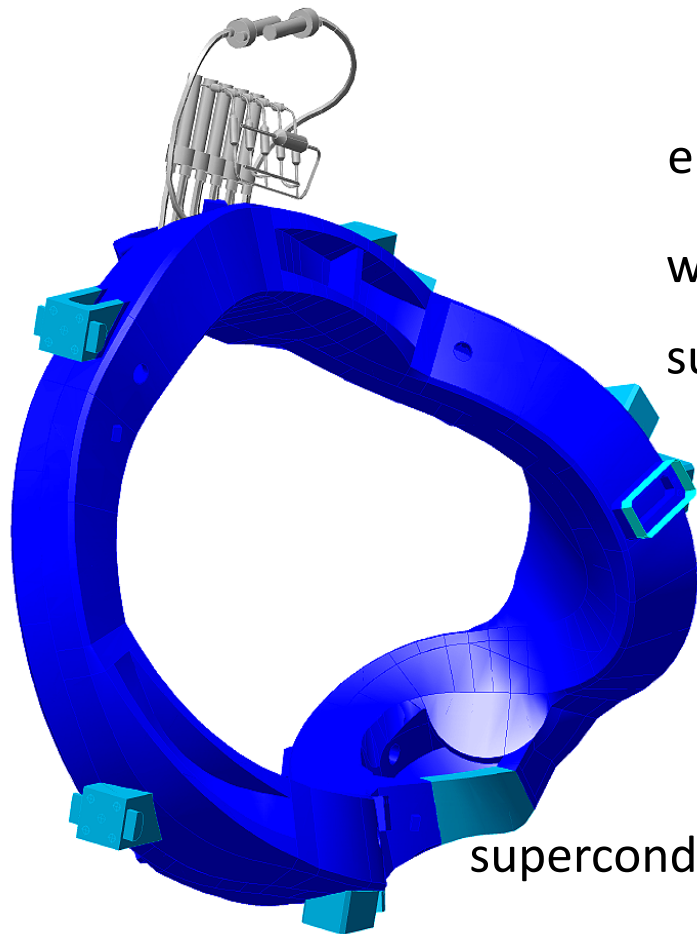
50 non-planar coils,
20 planar coils,
NbTi superconductors
($T < 3.4$ K)

induction on axis:
2.5 T

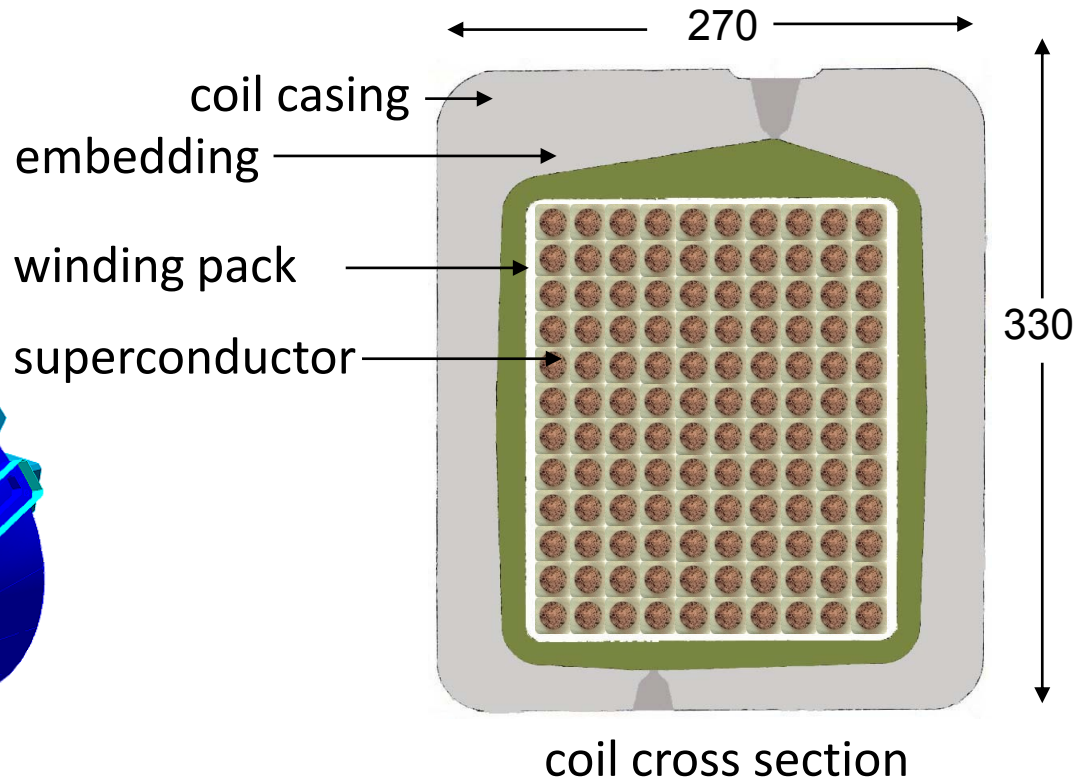
induction within coil:
6.8 T at 17.8 kA

source: IPP

non-planar superconducting coil



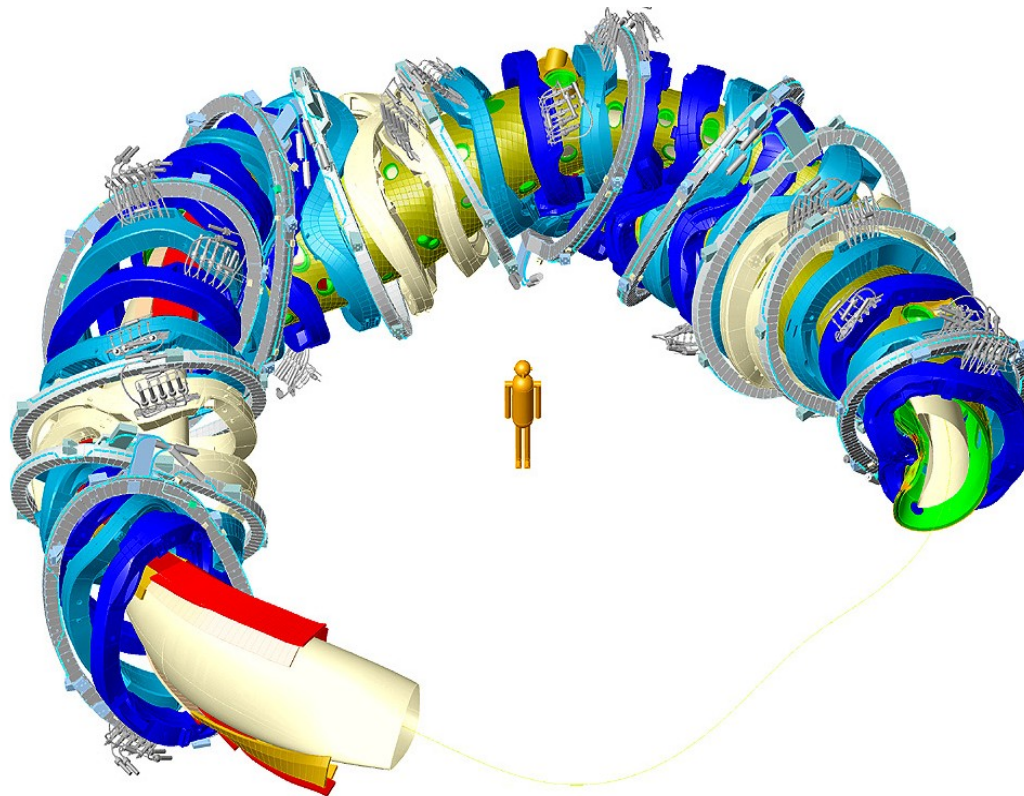
NbTi-superconductor ($T_C \approx 4$ K)



superconductivity: electric resistivity vanishes
below a transition temperature T_C

source: IPP

5 modules with 10 non-planar and 4 planar coils each



plasma
divertor
plasmavessel

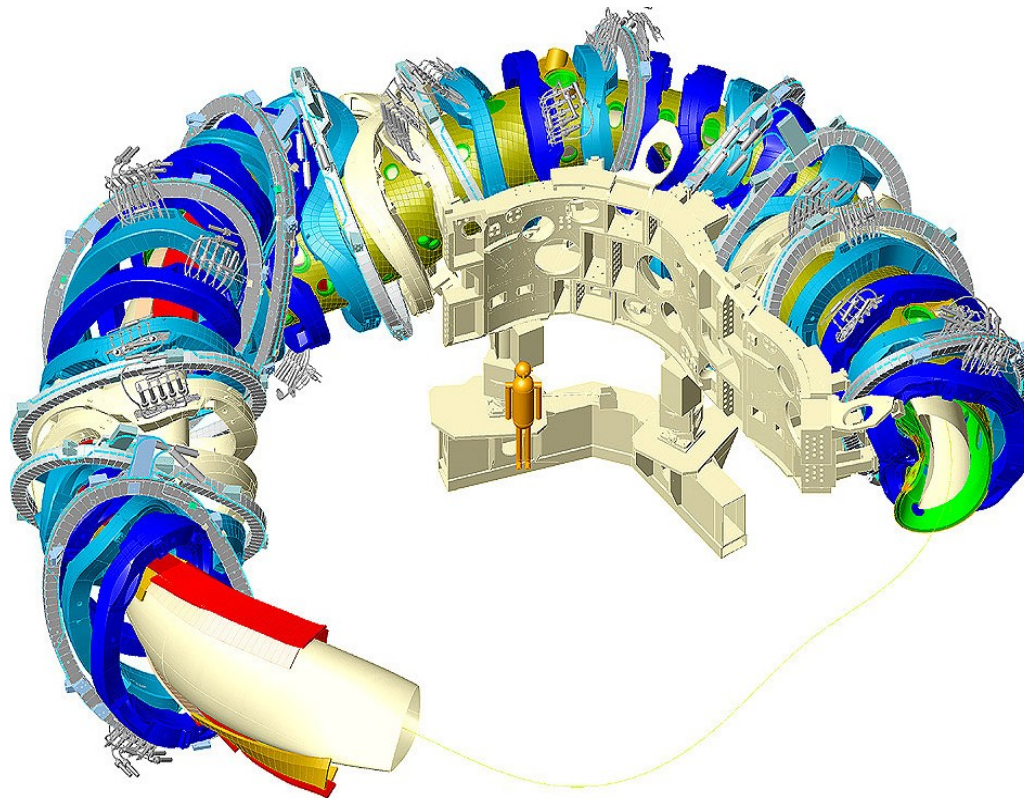
coil system

50 non-planar coils,
20 planar coils,
NbTi superconductors
($T < 3.4$ K)

induction on axis:
2.5 T

induction within coils:
6.8 T at 17.8 kA

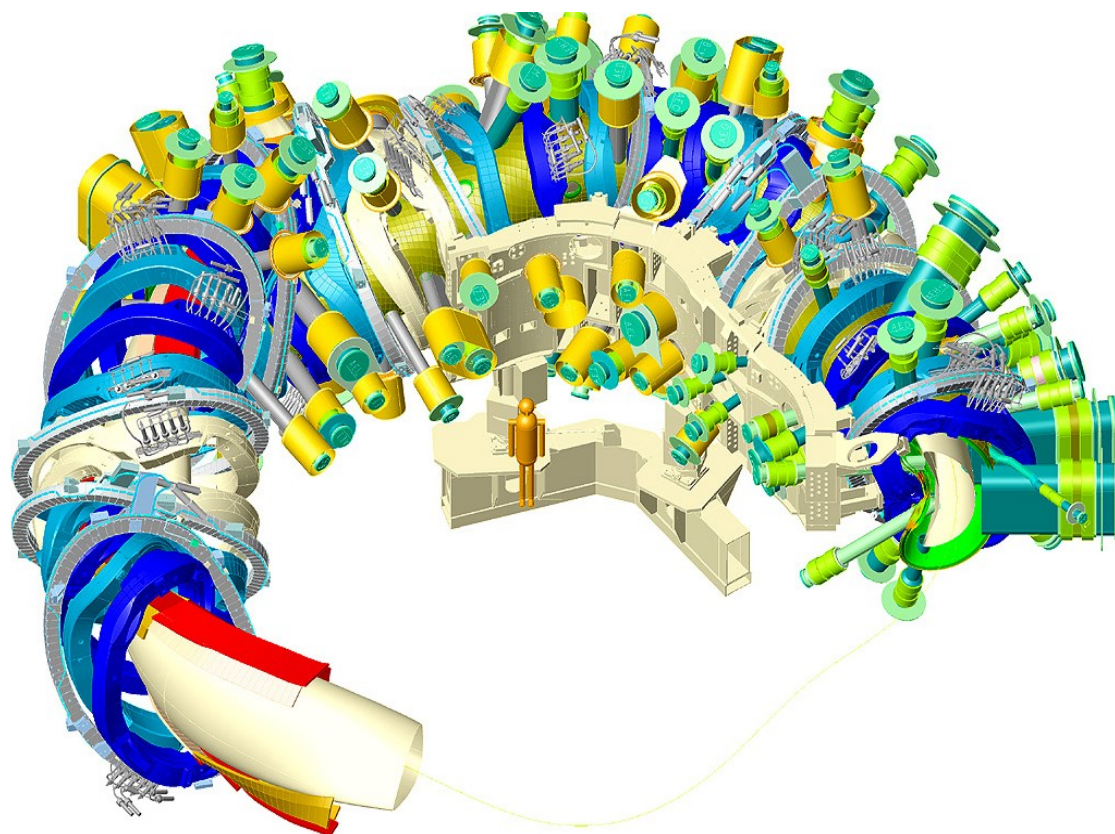
source: IPP



plasma
divertor
plasma vessel
coil system

support structure
max. force/coil: 3.6 MN

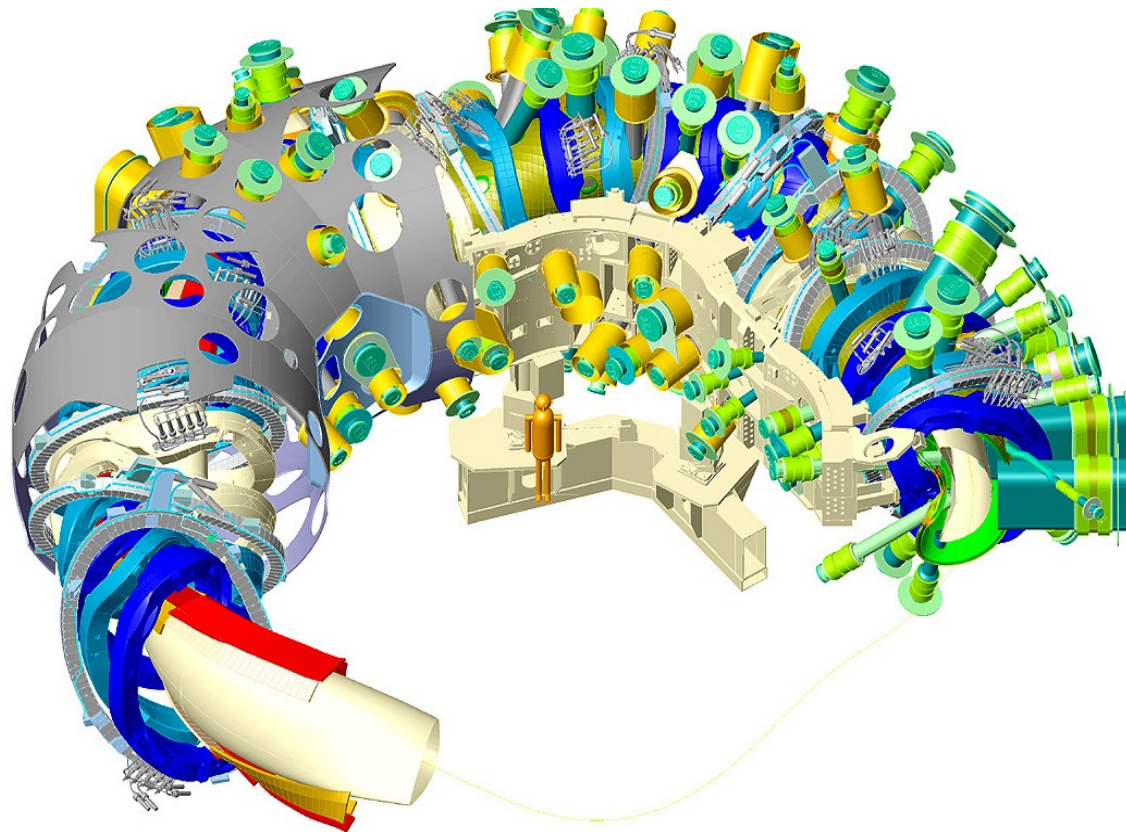
Quelle: IPP



plasma
divertor
plasma vessel
coil system
support structure

ports
254 ports
32 types

source: IPP

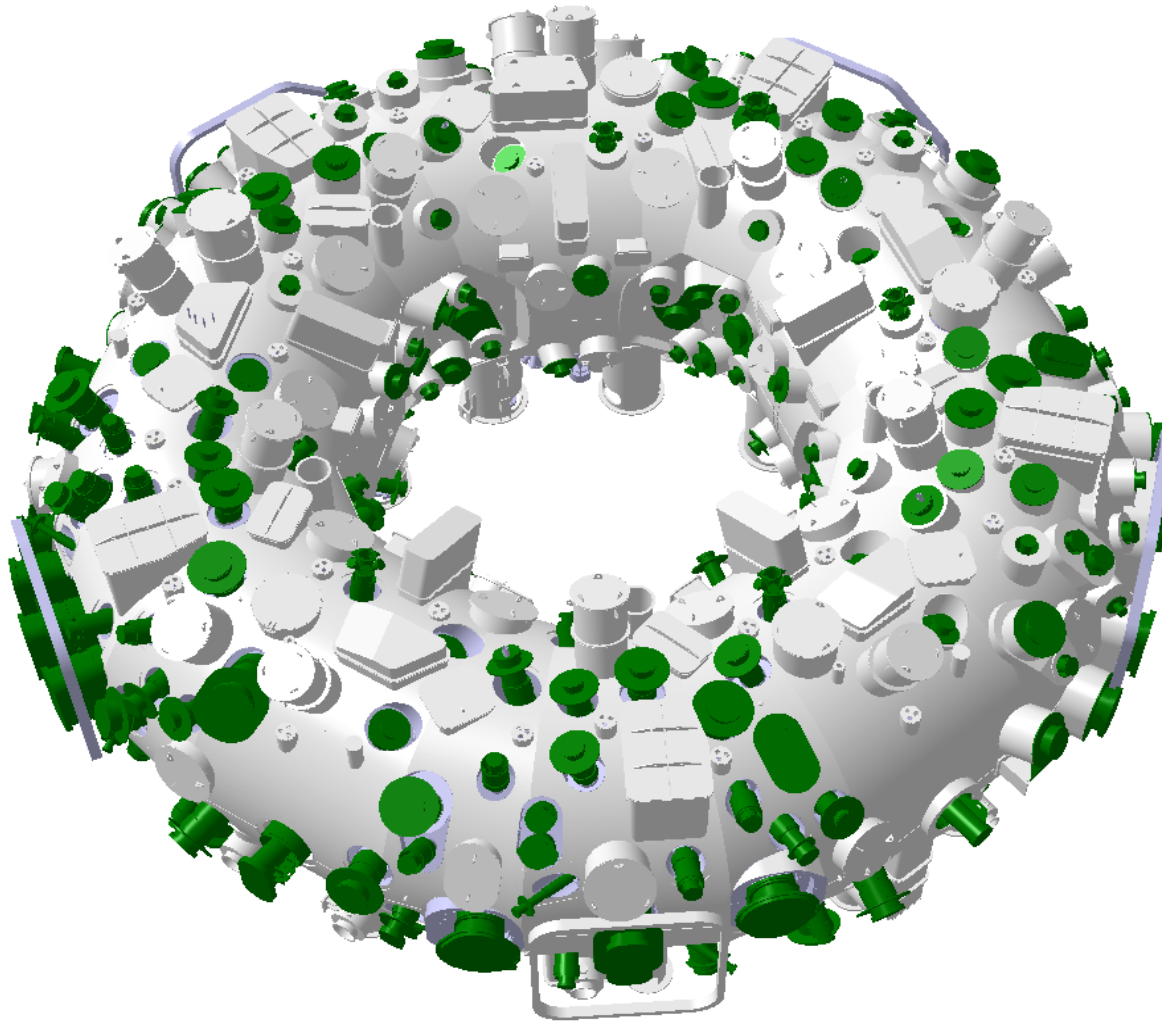


plasma
divertor
plasma vessel
coil system
support structure
ports

cryostat

volume: 525 m³
surface: 480 m²
vacuum: < 10⁻⁵ mbar
mass: 150 t

source: IPP

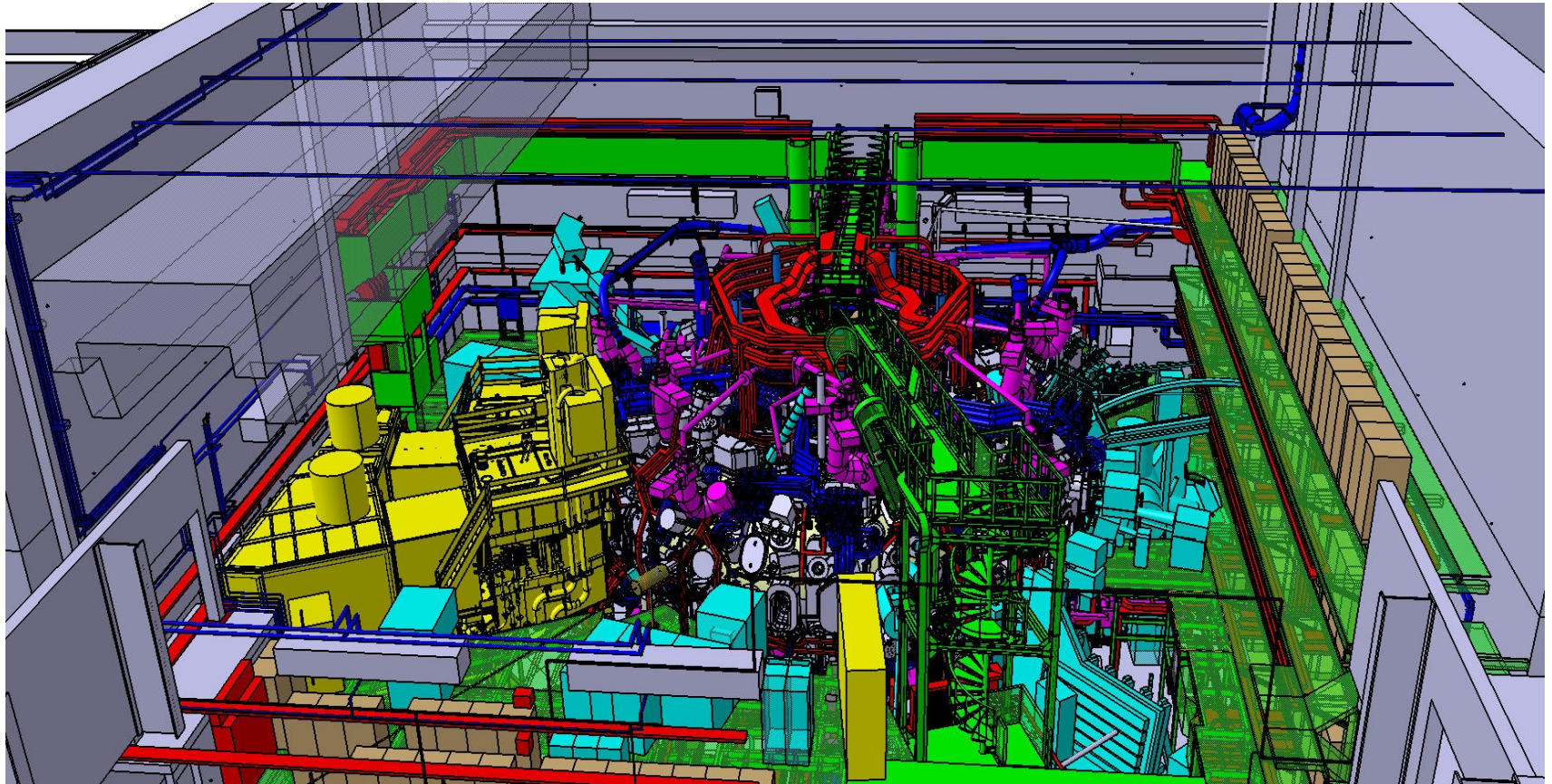


plasma
divertor
plasma vessel
coil system
support structure
ports
cryostat









W7-X

height: 4.5 m
diameter: 16 m
mass: 725 t
cold mass: 425 t
heating power: 15-30 MW
pulse duration: 30 min

source: C. Klug 2008



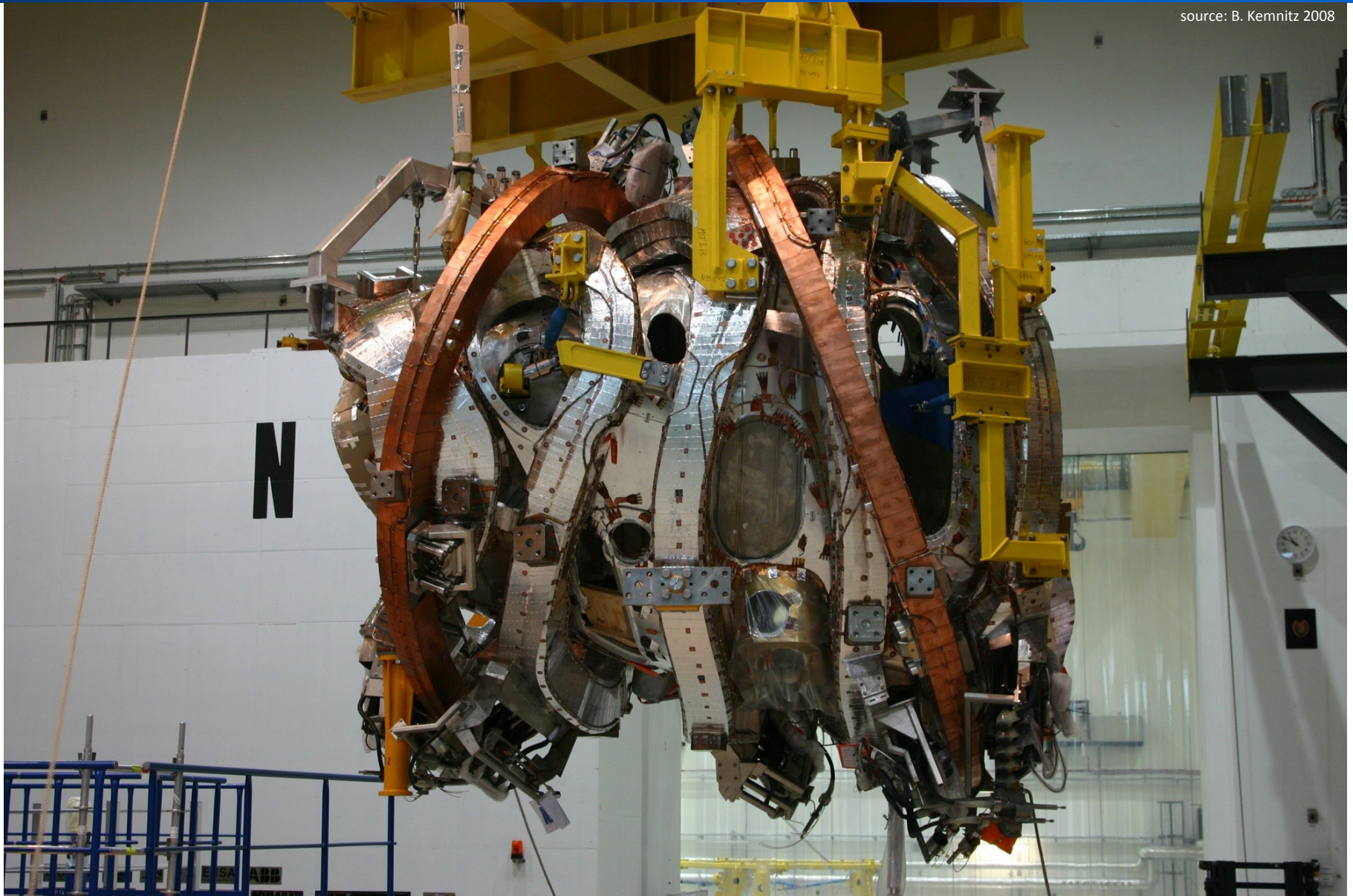
source: IPP

- | | |
|---|--|
|  torus hall with cryostat |  vacuum systems |
|  piping (cooling, extinguishing system, gas exhaust) |  electronic racks |
|  cable lines |  heating systems (without ECRH) |
|  Thomson bridge, heavy weight structure etc. |  several diagnostic modules |

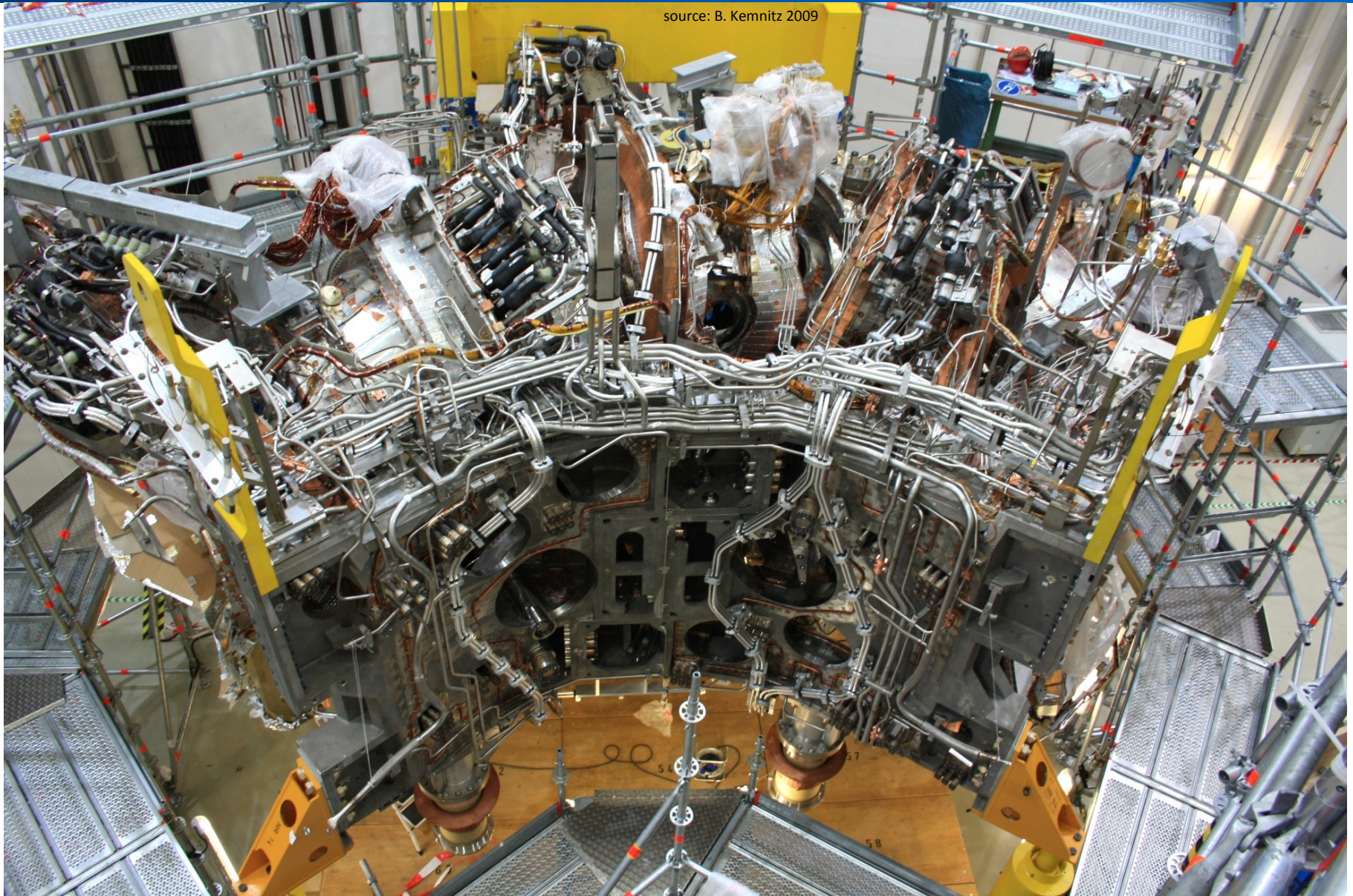
source: B. Kemnitz 2007



source: B. Kemnitz 2008

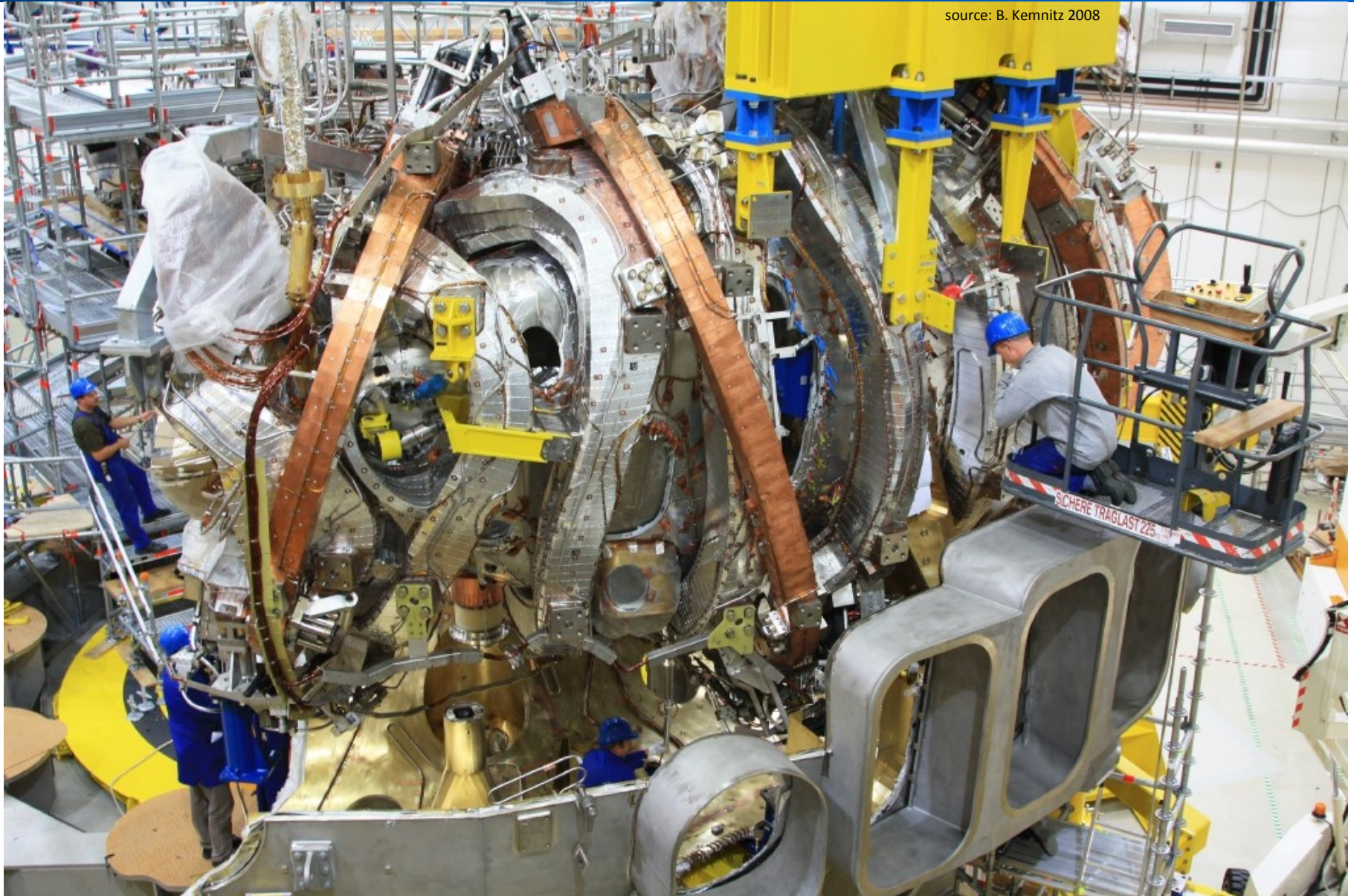


source: B. Kemnitz 2009

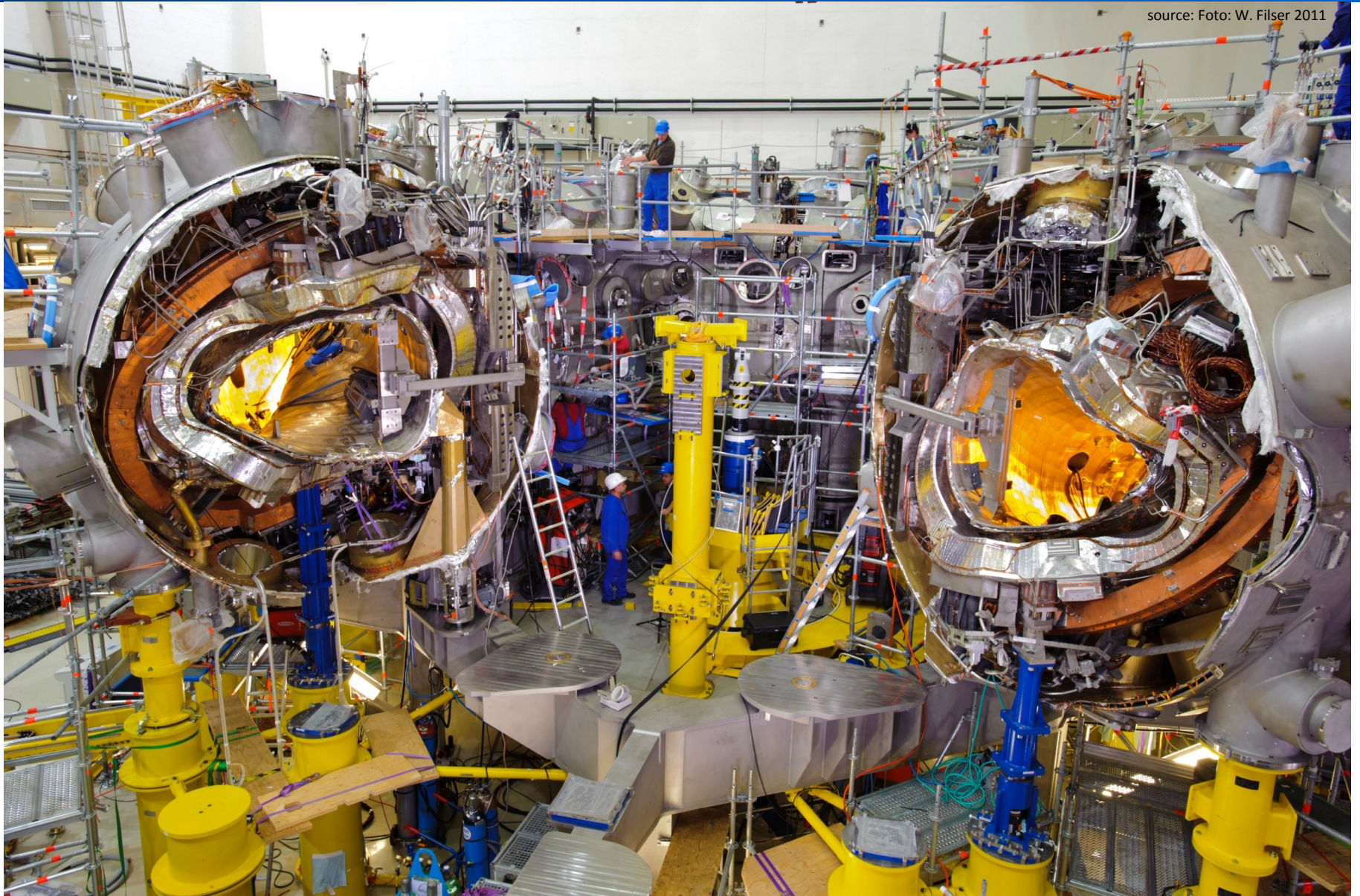


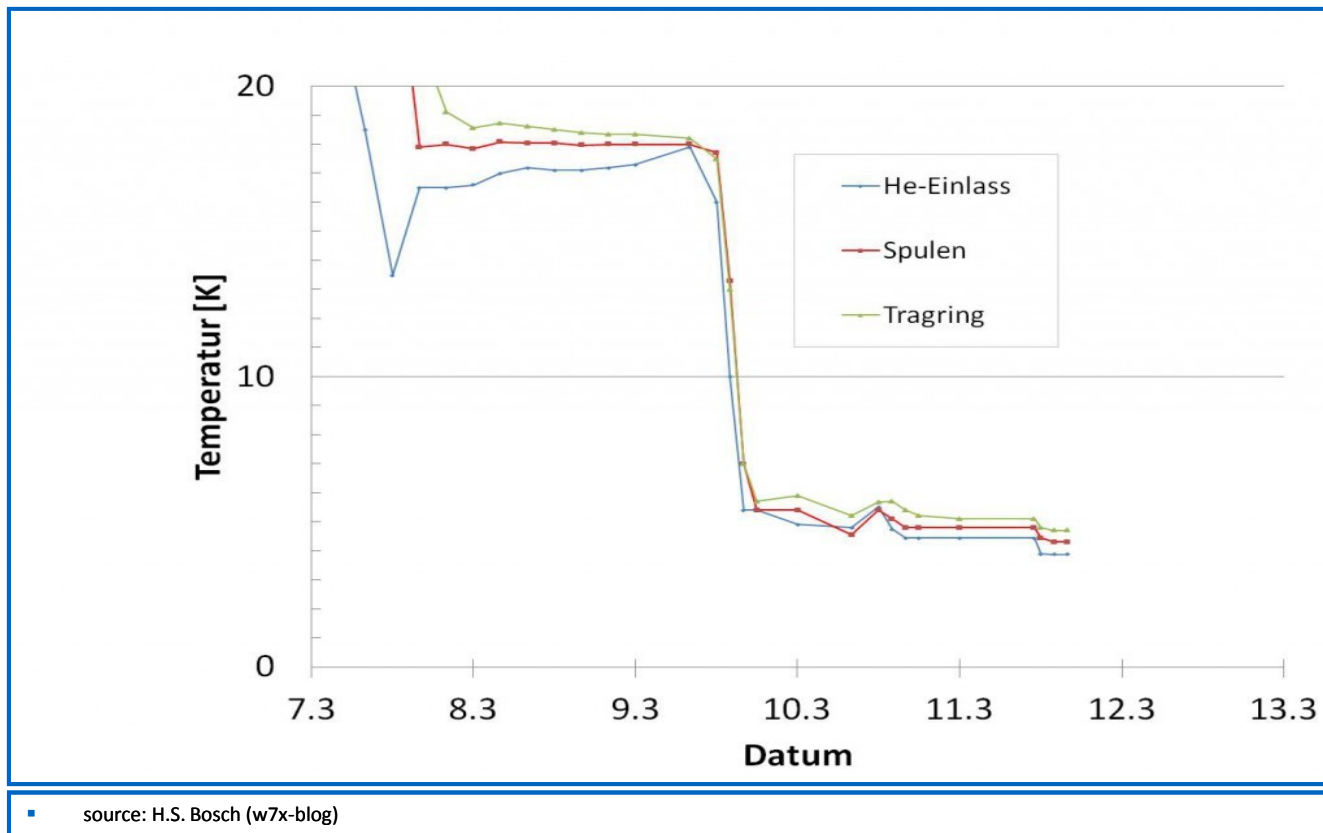
Magnetic Module with Lower Half of Cryostat

source: B. Kemnitz 2008

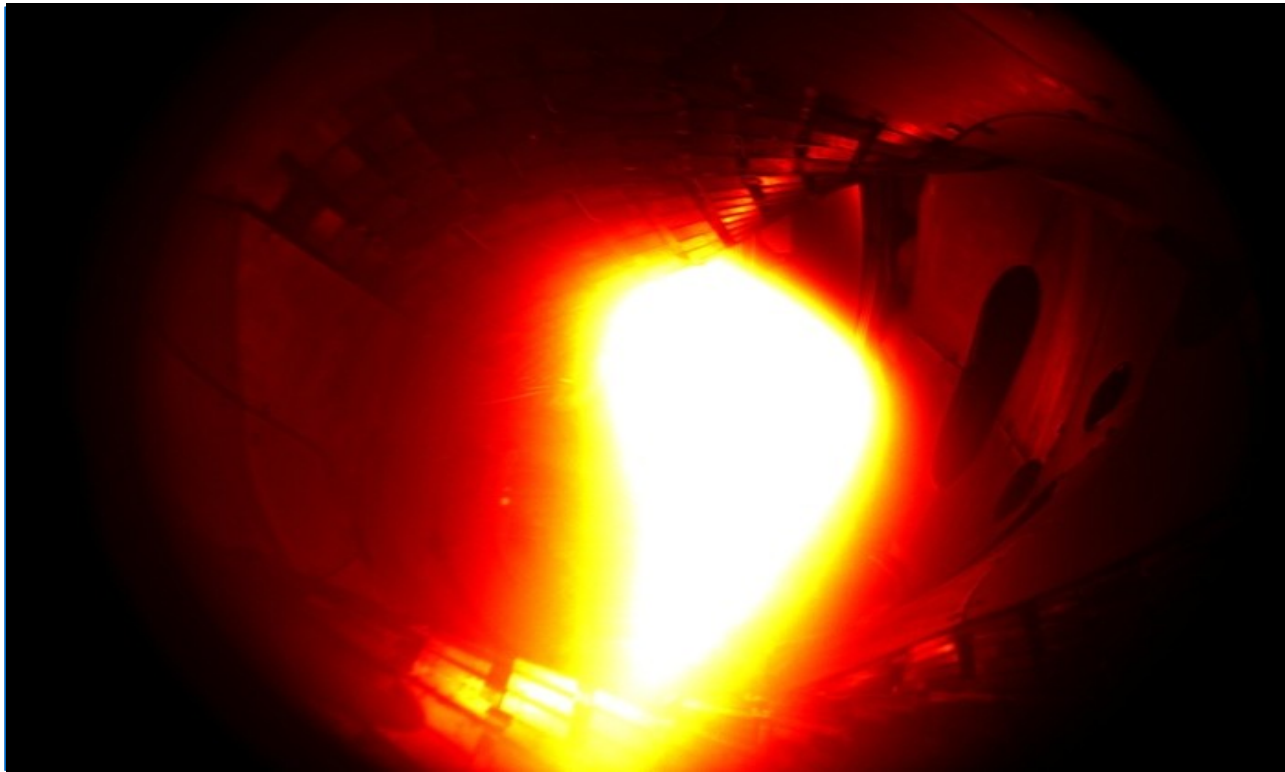


source: Foto: W. Filser 2011





- successful cool-down of cryostat (III/2015)
- successful test of magnetic system (VI/2015)
- successful evacuation of plasma vessel (VI/2015)

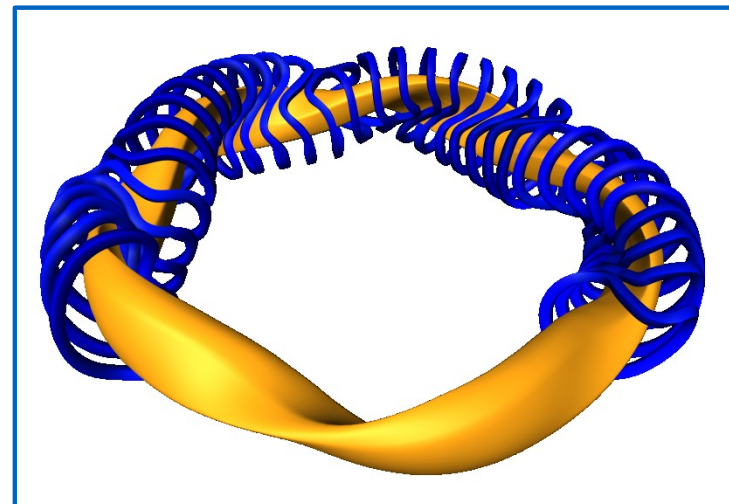


▪ source M. Otte (IPP)

- first flux surface measurements (VII/2015)
- bake-out of plasma vessel (IX/2015)
- first plasma (XII/2015)

Time-lapse movie of assembly

- confirm:
 - feasibility (coil system)
 - successful optimisation
 - long-pulse operation
(suitability for power plant)
- extensive research programme:
 - transport of plasma and impurities
 - efficiency of non-ohmic heating (ECRH, ICRH, NBI)
 - control of boundary layer plasmas (divertor)
 - verification of numerical models

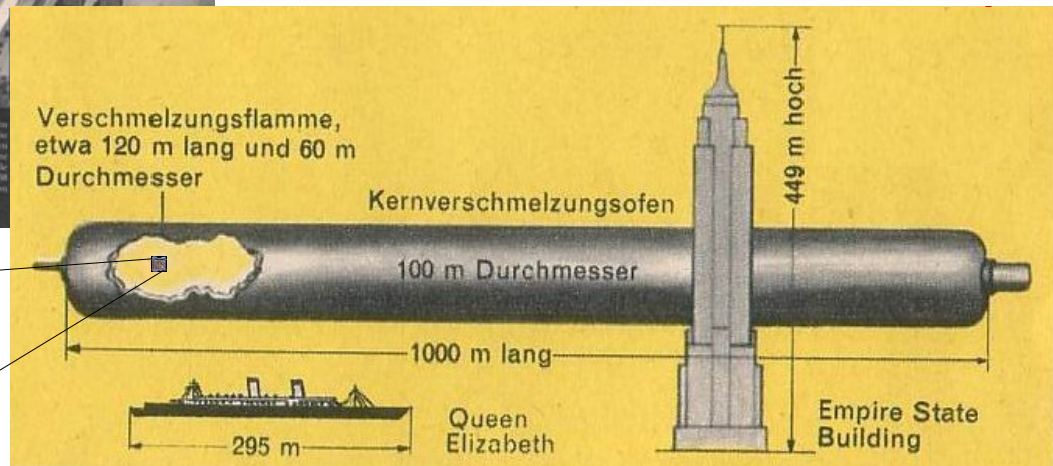
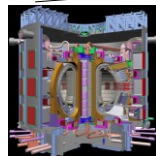


Quelle: M. Borchardt 2012

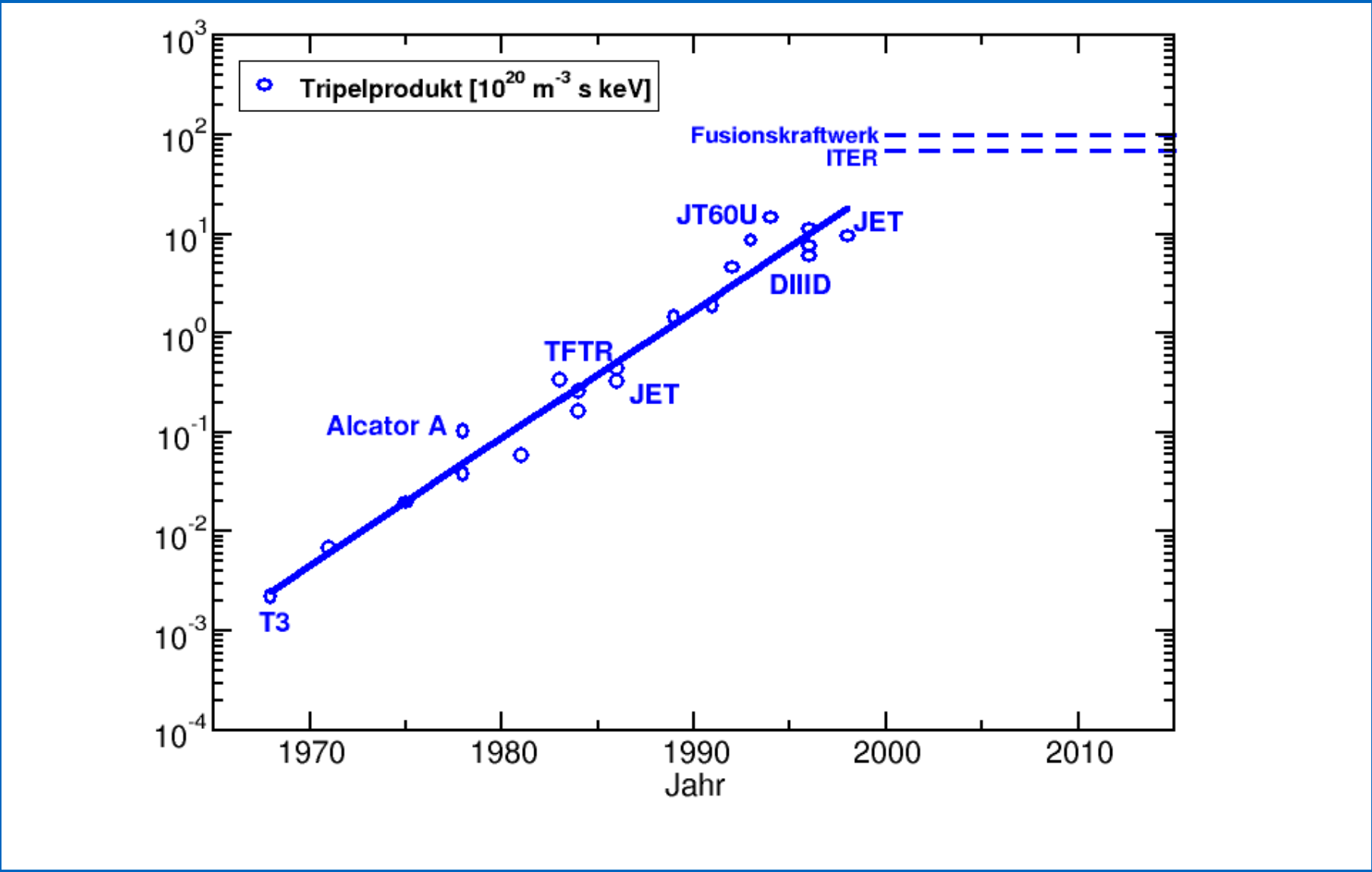
W7-X is an experiment and will not produce any energy
(HH and HD plasmas only)

Do stellarators play a role for future power plants?

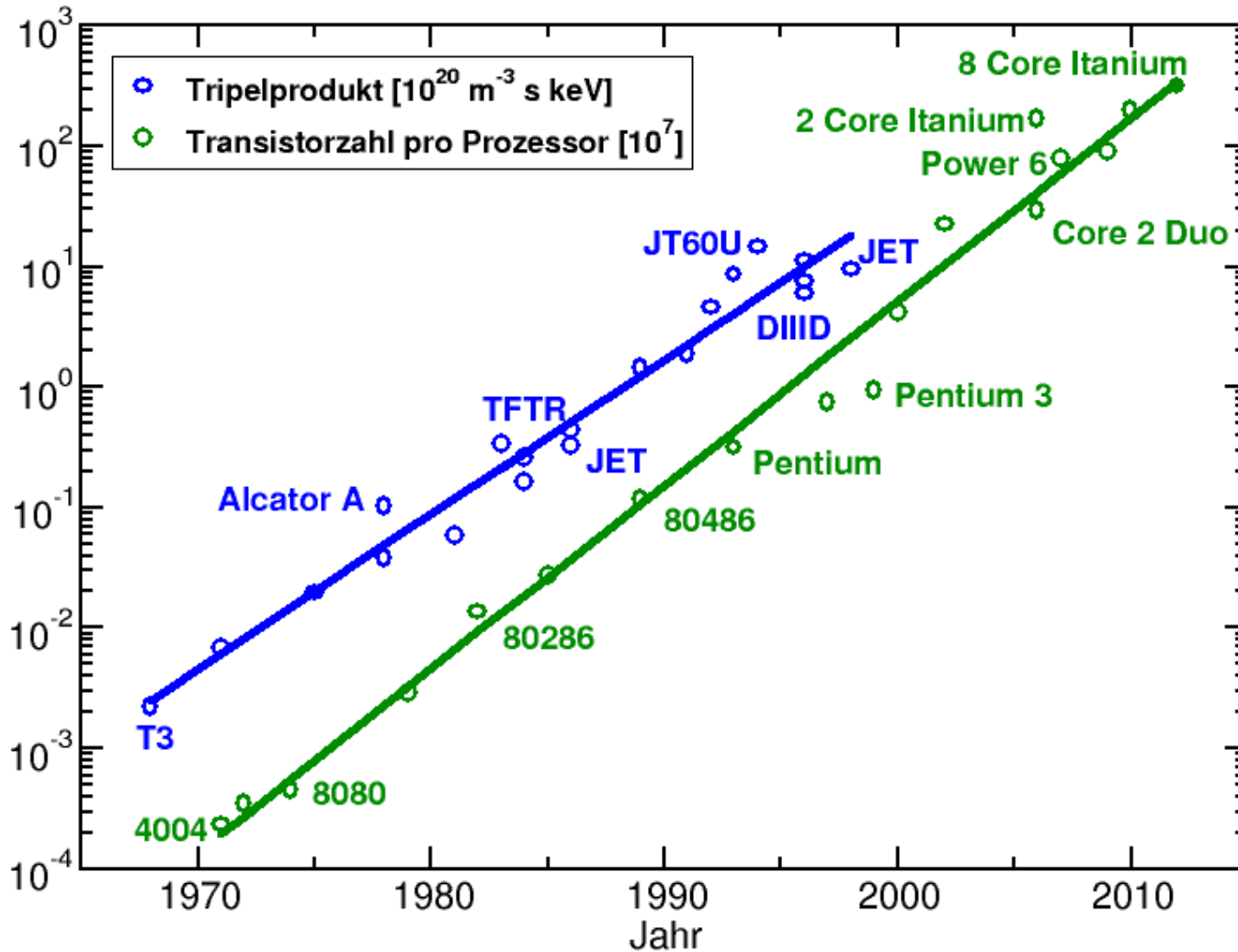
SUMMARY & OUTLOOK



source: article in "Hobby", 1957, Nr. 8, S. 68



Quelle: R. Kleiber 2012



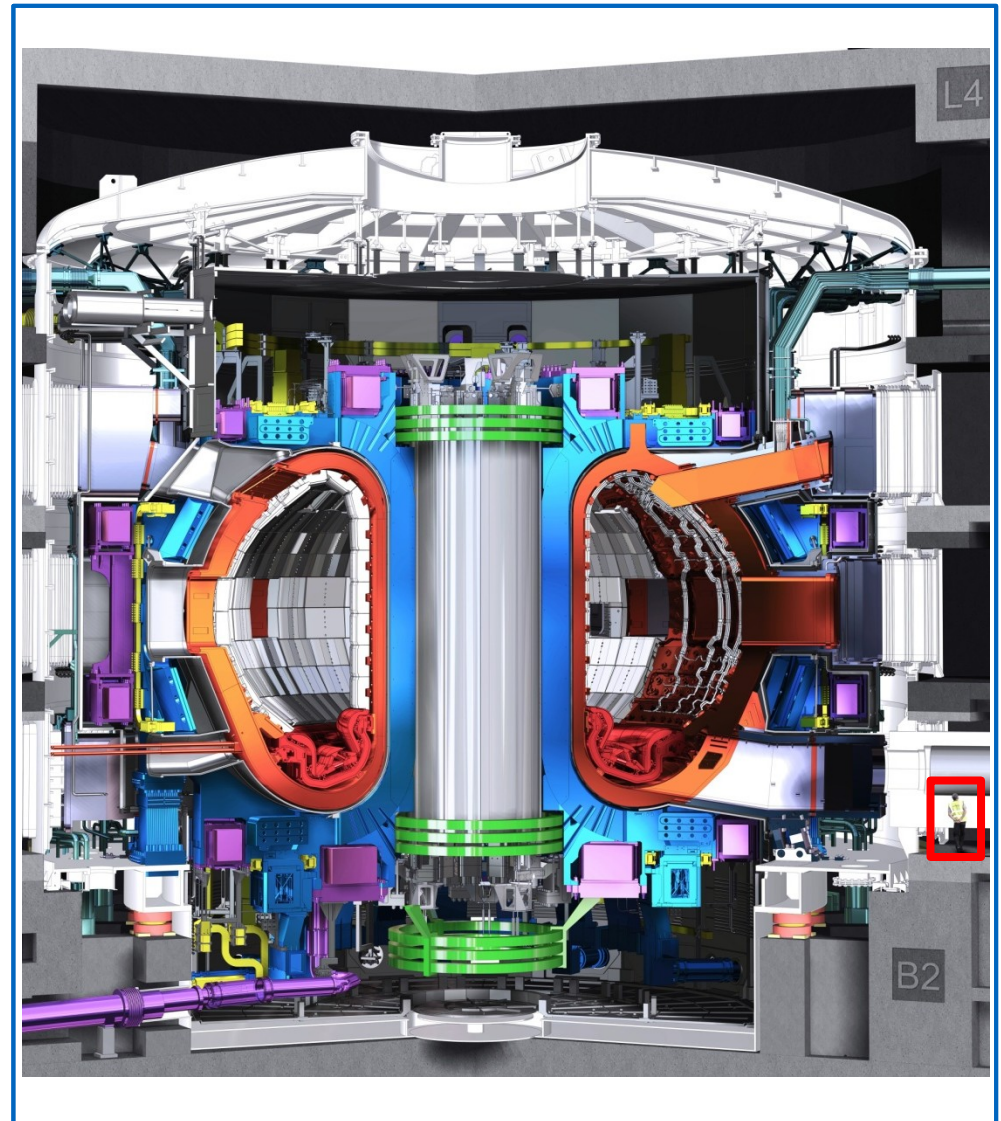
Quelle: R. Kleiber 2012

- final report: 2001
- site: CEA Cadarache
- start of construction: 2009
- first plasma: 2025

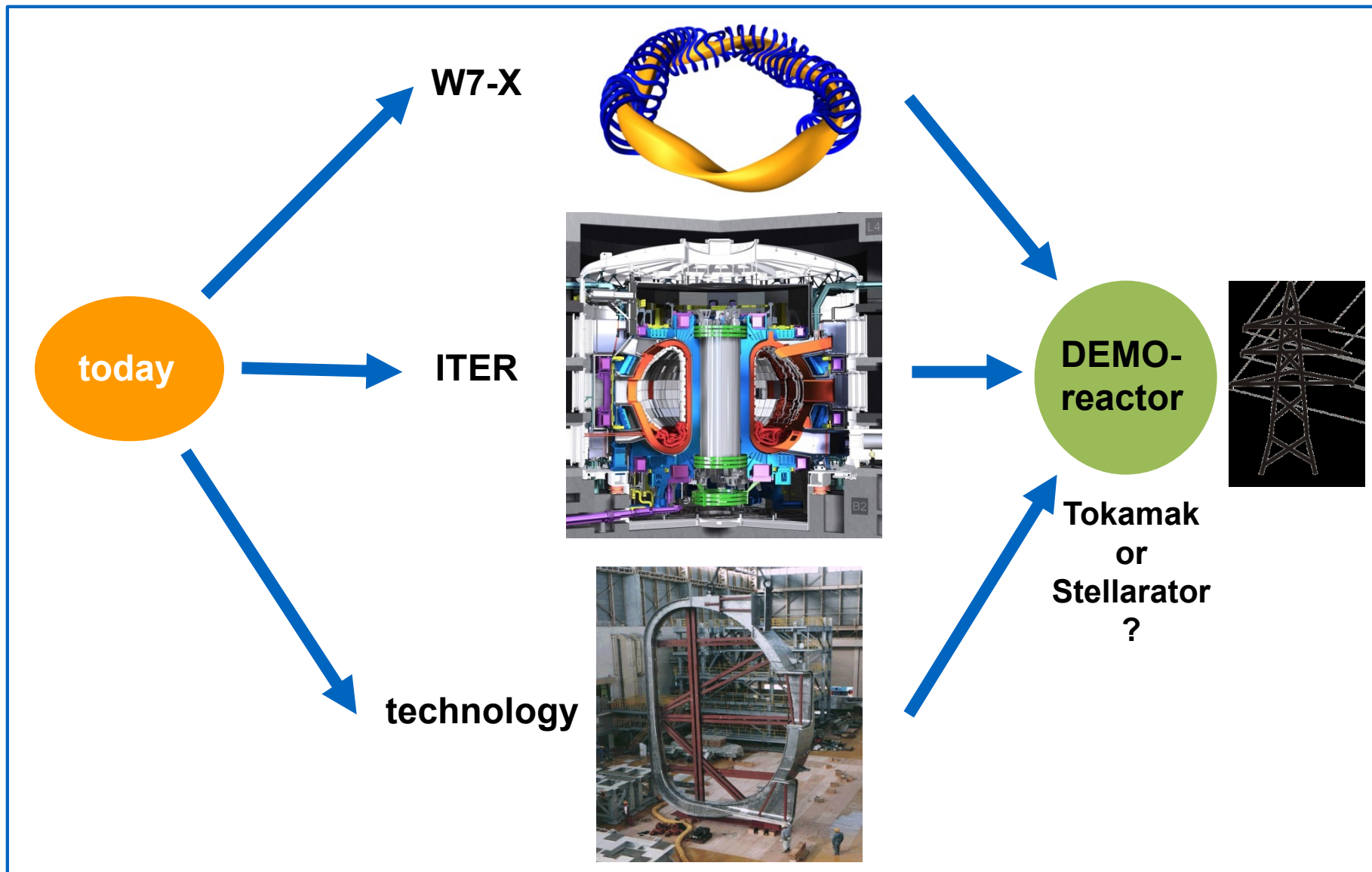
The purpose of ITER is to show the scientific and technological feasibility of fusion power.

basic specifications

| | |
|--------------------------|-----|
| R [m] | 6.2 |
| a [m] | 2.0 |
| T_{Puls} [s] | 300 |
| P_{Fusion} [MW] | 500 |
| power gain (Q) | 10 |
| cost [G€] | 15 |



Quelle: ITER 2013



Tokamaks

- advanced operational scenarios, i.e. current drive
- suppression of disruptions
- control of edge localised plasma bursts (ELMs): 10 GW/m² for ITER

Stellarators

- suitability of the stellarator for power generation
- impurity transport and confinement of α -particles

Materials and Technology

- divertor materials: high heat fluxes (10 MW/m²)
- wall materials: fatigue caused by strong neutron flux
- robotic maintenance technology (remote handling)

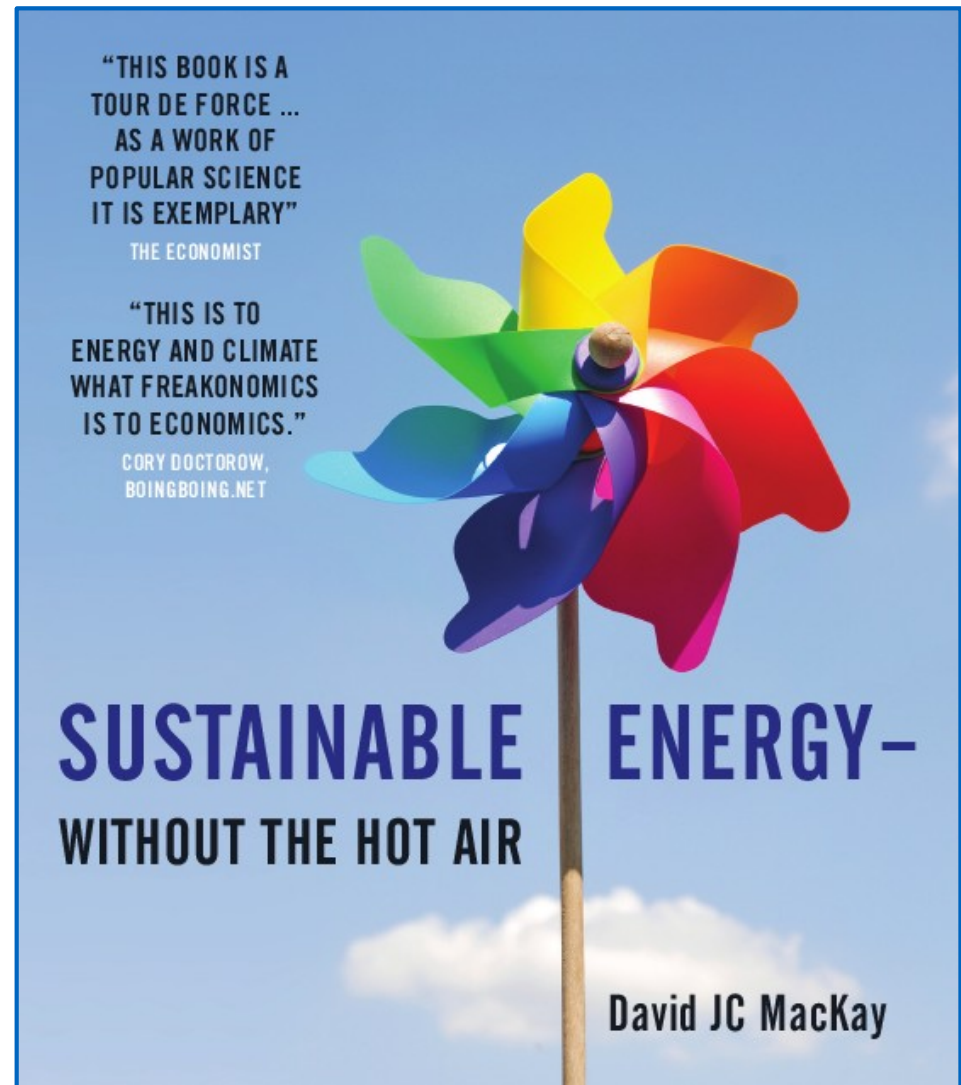
Fuel Cycle

- development of blanket technology
- breeding ratio of 1.1 -1.15 required
- implementation of Tritium cycle
- exhaust of helium ash

- Nuclear fusion is a virtually inexhaustible source of energy.
- Fusion power offers an enhanced level of safety that is due to its fundamental physical properties.
- Nuclear fusion is achieved in sufficiently hot and dense plasmas.
- Magnetic confinement is an approach to utilise fusion energy.
- Experiments explore the potential of both tokamaks and stellarators.
- W7-X is an optimised stellarator designed for operation with long pulses.
- ITER is expected to be the first machine generating excess power.
- DEMO will be the first fusion power plant.

David J.C. MacKay,
UIT Cambridge (2009)
www.withouthotair.com

free download
(in many languages)



source: <http://www.withouthotair.com/> 2009

- advanced training for teachers on „Generation and Consumption of Energy“
- guided tours at the Max-Planck-Institut für Plasmaphysik in Greifswald and Garching

Contact

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E-Mail: besucher.greifswald@ipp.mpg.de

Garching: Ms. I. Kaufmann, Boltzmannstraße 2,
D-85748 Garching, Tel.: 089-3299-2233 (Mo - Do, 8:30 - 12 Uhr),
Fax: 089-3299-2622, E-Mail: besucher.garching@ipp.mpg.de